Light Water Reactor Sustainability Program

Flexible Plant Operation and Generation: Hazards and Probabilistic Risk Assessments of a Light-Water Reactor Coupled with Industrial Facilities



September 2024

U.S. Department of Energy

Office of Nuclear Energy

DISCLAIMER

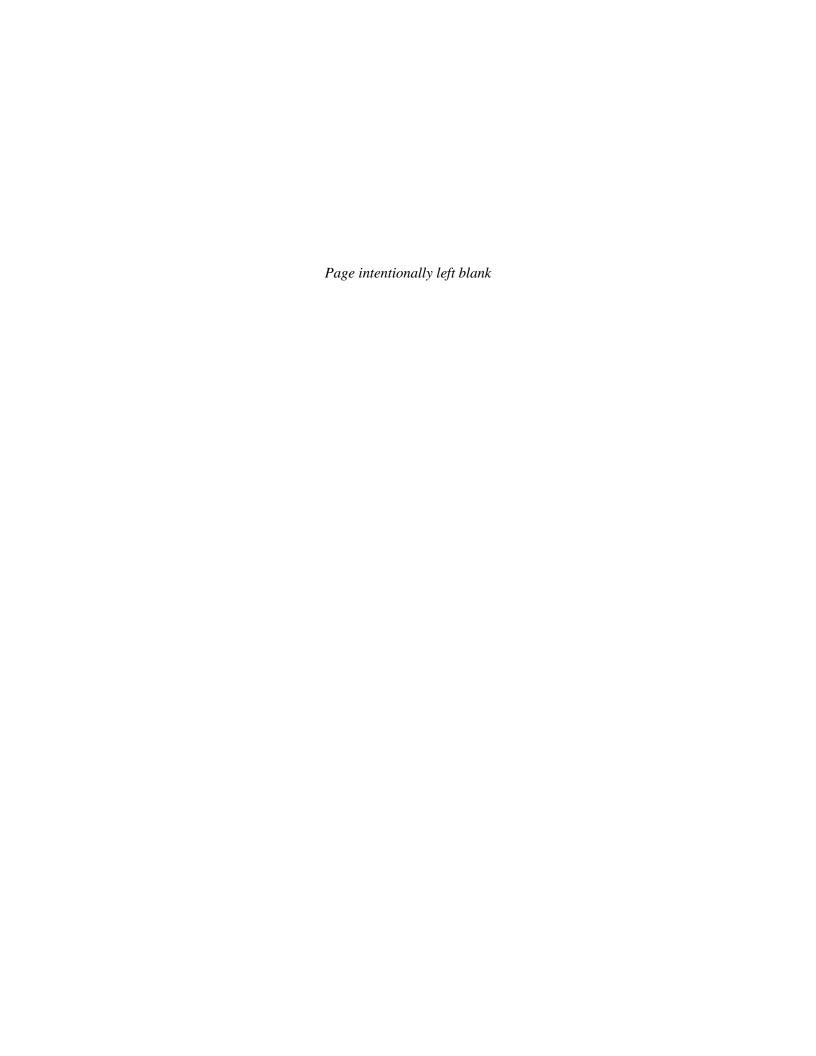
This information was prepared as an account of work sponsored by an agency of the U.S. Government. Neither the U.S. Government nor any agency thereof, nor any of their employees, makes any warranty, expressed or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness, of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. References herein to any specific commercial product, process, or service by trade name, trade mark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring by the U.S. Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the U.S. Government or any agency thereof.

Hazards and Probabilistic Risk Assessments of a Light-Water Reactor Coupled with Industrial Facilities

Kurt G. Vedros Robby Christian Courtney Otani Wen-Chi Cheng Ronald Gonzales

September 2024

Prepared for the U.S. Department of Energy Office of Nuclear Energy



EXECUTIVE SUMMARY

This report provides a roadmap and toolkit for site-specific risk assessments across a broad range of industrial customers co-located with nuclear power plants (NPPs). This report builds upon the body of work sponsored by the Department of Energy (DOE) Light-Water Reactor Sustainability (LWRS) Flexible Plant Operation and Generation Pathway that presented hazards assessment and generic probabilistic risk assessments (PRAs) for the addition of a heat extraction system (HES) to light-water reactors co-located with hydrogen production facilities [1]. The report expands the hazards assessments to include other industrial facilities: an oil refinery, a methanol plant, a synthetic fuel (synfuel) plant, the production of synthetic gas (syngas) as part of the methanol and synfuel plants, and wood pulp and paper mills [2]. All these facilities are specified through industrial process and requirements research performed by national laboratories, universities, and interaction with industry. Many of the processes used in this report are pre-conceptual designs to use for decarbonization of the current technology facilities. A process of failure modes and effects analysis (what can go wrong) and accidentology (what has historically gone wrong) was used to determine the hazards presented to the NPP by the addition of the HES and the industrial customer. Chemical properties of feedstocks and products are summarized as part of the hazards assessment. Example analysis procedures are provided for each of the hazard types identified. These deterministic analyses can be used to assess adherence to licensing criteria. They can also be used to meet other safety goals like protection of the public, workers, or industrial facility equipment. The probabilistic analysis consisted of three sizes of HESs modeled in a PRA to assess the impact on the initiating events (IE) and results of the PRA. The PRA results conclude that the resulting increases in IE frequencies are below the limits required for small changes to existing NPPs under 10 CFR 50.59.

Page intentionally left blank

ACKNOWLEDGEMENTS

The authors of this report would like to thank Austin Glover and the HyRAM+ team at Sandia National Laboratories for outstanding support in the hydrogen plant safety analysis leak rates, frequencies, and overpressure estimates.

We would also like to thank Jack Cadogan, Jeffrey Brown, and Jason Remer for their great support both through the interaction with DOE's LWRS Hydrogen Regulatory Research and Review Group and for advice and guidance from the intended audience's perspective.

Page intentionally left blank

CONTENTS

EXE	ECUTI	VE SUM	MARY	iii		
ACI	KNOW	LEDGE	MENTS	v		
ACI	RONYI	MS		XV		
1.	INTF	INTRODUCTION				
	1.1	Nuclea	ar-Supported Industrial Facilities	1		
	1.2	Risk A	Assessment Roles in Safety and Licensing of Nuclear Power Plant Modifications	2		
2.	PRO.	JECT SO	COPE	3		
3.	SPEC	CIFICAT	TIONS OF THE SUPPORTED INDUSTRIAL FACILITIES	4		
	3.1	High T	Cemperature Electrolysis Hydrogen Facility	4		
	3.2		nol Plant			
	3.3	Synthe	etic Fuel Production	5		
		3.3.1	Intermediary Methanol Product in Synthetic Fuel Plant			
		3.3.1	Fischer-Tropsch Process in Synthetic Fuel Plant			
	3.4	Petrole	eum Refinery	6		
	3.1	Pulp a	nd Paper Mill	6		
4.	NUC	LEAR F	OWER PLANT MODIFICATIONS FOR AN INDUSTRIAL CUSTOMER	7		
	4.1	Nuclea	ar Power Plant with Heat Extraction System	7		
		4.1.1	Low Level Heat Extraction System Design (≤ 100 MW _t)	8		
		4.1.2	Mid-Level Heat Extraction System Design (100 MW $_t$ to \leq 200 MW $_t$)	10		
		4.1.3	High-Level Heat Extraction System Design (> 200 MW _t)	12		
	4.2	Direct	Electrical Connection	15		
5.			NALYSIS OF A NUCLEAR POWER PLANT SUPPLYING ENERGY TO AN L FACILITY			
	5.1	Accide	entology	19		
		5.1.1	Hydrogen Electrolysis	19		
		5.1.2	Methanol Plant Accidentology	22		
		5.1.3	Syngas Production Accidentology	23		
		5.1.4	Oil Refinery Accidentology	23		
		5.1.5	Pulp and Paper Mill Accidentology			
	5.2	_	Options and Assumptions			
	5.3	Failure	e Modes and Effects Analysis	26		
		5.3.1	Nuclear Power Plant Hazards Analysis	27		
		5.3.2	Methanol Plant Hazards for use in FMEA			
		5.3.3	Syngas Production Hazards for use in FMEA			
		5.3.4	Refinery Plant Hazards for use in FMEA	31		

		5.3.5 Pulp and Paper Mill Hazards for use in FMEA	32
	5.4	Nuclear Power Plant Safety-Critical Structures	33
		5.4.1 Reactor Containment Structure Fragility to Overpressure Events	33
		5.4.2 Safety-Critical External Structures Fragility to Overpressure Events	
		5.4.3 Non-Safety-Critical External Structures	34
		5.4.4 Heat Extraction System Unisolable Steam Pipe Rupture	34
		5.4.5 Heat Extraction System Reboiler Leak	35
		5.4.6 Prompt Steam Diversion Loss Causes Feedback	
		5.4.7 Use of Heat Transfer Fluids and Ignition Potential	
	5.5	Industrial Facility Siting Analysis	36
		5.5.1 Blast Analysis	
		5.5.2 Blast Mitigation Strategies	
		5.5.3 Atmospheric Dispersion Analysis	
	5.6	Analysis of Heat Flux from Fires and Fireballs	49
	5.7	General Plant Transient Due to Overcurrent from Electrical Transmission	53
	5.8	Control Room Habitability	54
6.	PRO	BABILISTIC RISK ASSESSMENT MODEL	56
	6.1	Electrical Transmission Probabilistic Risk Assessment Model	56
	6.2	Generic Pressurized Water Reactor Model	62
		6.2.1 Heat Extraction System Linkage into the Pressurized-water Reactor Model	64
	6.3	Generic Boiling Water Reactor Model	68
		6.3.1 Heat Extraction System Linkage into the Boiling Water Reactor Model	69
7.	RESU	ULTS OF PROBABILISTIC RISK ASSESSMENT	72
	7.1	Nominal Probabilistic Risk Assessment Results	72
		7.1.1 Pressurized-water Reactor Probabilistic Risk Assessment Results	73
		7.1.2 Boiling Water Reactor Probabilistic Risk Assessment Results	74
8.	LICE	ENSING PATHWAY SUPPORT FROM PROBABILISTIC RISK ASSESSMENT	75
	8.1	Licensing Process through 10 CFR 50.59	76
	8.2	Adherence to the Site Fire Protection Plan	77
	8.3	Licensing Support through RG 1.174	78
	8.4	Licensing Support through RG 1.91	79
	8.5	Licensing Amendment Review Process	79
9.	CON	CLUSIONS	80
10.	REFI	ERENCES	81
App	endix A	A Generic PWR PRA Model	88
App	endix I	3 Generic BWR PRA Model	96
Δnn	endiv (TEMEA Criteria	112

Appendix D FMEA Results for SynGas Production	. 114
Appendix E: FMEA Results- Methanol Synthesis Facility	. 124
Appendix F: FMEA Results- Petroleum Refinery Facility	. 132
Appendix G: FMEA Results- Pulp and Paper Facility	. 166
Appendix H: Industrial Products and Feedstock Physical Properties for Safety Analysis Supporting Information	. 172
Appendix I: Gas Component Leakage Frequencies for Safety Analysis Supporting Information	. 177
FIGURES	
Figure 1-1. Nuclear can provide heat and electricity for many industrial processes.	1
Figure 3-1. Synthetic fuel production process flow, highlighting the final stages that are similar to a traditional oil refinery [7]	5
Figure 4-2. \leq 100 MW _t industrial customer diagram of steam extraction piping to the reboiler [6]	10
Figure 4-3. Model drawing of mid-level HES (> $100~\text{MW}_t$ to $\leq 200~\text{MW}_t$) industrial facility requirement.	11
Figure 4-4. 100 MW_t to $\leq 200 \text{ MW}_t$ industrial customer diagram of steam extraction piping to the reboiler [6]	12
Figure 4-5. Model drawing of high-level HES (>200 MW _t).	14
Figure 4-6. $> 200 \text{ MW}_t$ industrial customer diagram of steam extraction piping to the reboilers	15
Figure 4-7. Power triangle diagram [12]	16
Figure 4-8. Transmission line and portion of ring bus switchyard arrangement at NPP [6]	18
Figure 4-9. Behind-the-meter physical layout of electrical feeder [6].	19
Figure 5-1. Hydrogen incident statistics per application type (top) and their causes (bottom) [16]	20
Figure 5-2. Hydrogen incident consequences per specific application supply chain stage [15]	21
Figure 5-3. Accident causes during hydrogen production [15]	22
Figure 5-4. Probabilistic consequences of syngas release.	23
Figure 5-5. Historical statistics of accidents in oil refineries: over time (left) and consequences (right).	24
Figure 5-6. Schematic representation of possible accidents following a loss of containment [35]	38
Figure 5-7. Example of a passive explosion isolation valve [43].	41
Figure 5-8. Example of an explosion suppression technique [44].	42
Figure 5-9. Blast barriers made of modular concrete blocks (left) [45] and metal (right) [46]	43
Figure 5-10. Illustration of a sacrificial blast wall [47].	44
Figure 5-11. Hierarchy of controls	44
Figure 5-12. Illustration of Gaussian dispersion model [49].	45

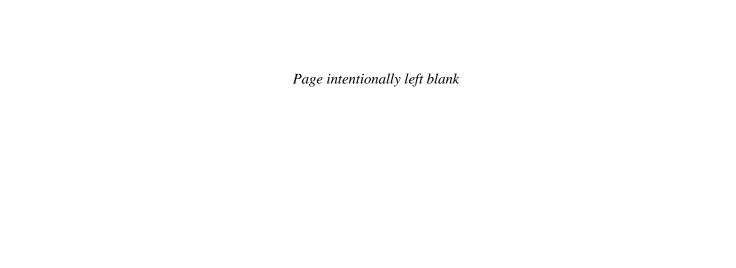
Figure 5-13. Illustration of heavy gas dispersion model [41].	46
Figure 5-14. 10-year wind speed rose graphs for the reference riverside site (left) and the reference desert site (right).	47
Figure 5-15. ALOHA output window showing syngas toxic areas for a given wind speed and direction, for the Gaussian dispersion model of flow #1 (left) and the heavy gas dispersion model of flow #2 (right)	48
Figure 5-16. Distances at level of concerns for the hypothetical riverside (left) and desert sites (right).	49
Figure 5-17. Geometry of a fireball and its distance to a target.	50
Figure 5-18. Comparison of fireballs resulting from the combustion of 1 gallon to 100,000 gallons of jet fuel	52
Figure 5-19. Heat flux from various synfuel product fireballs.	53
Figure 5-20. Transient syngas concentration in the control room of a nuclear power plant	55
Figure 6-1. Overall FT (IE-TRANS-HES1).	57
Figure 6-2. Breaker in Industrial Facility (IE-TRANS-HES100).	58
Figure 6-3. Breaker in Plant Boundary (IE-TRANS-HES101).	58
Figure 6-4. Breaker for Generator (IE-TRANS-HES102).	59
Figure 6-5. Overcurrent by industrial facility transformer or load expanded trees (IE-TRANS-HES103).	59
Figure 6-6. Example of a branch in Industrial Facility Load Failure: Branch A.	60
Figure 6-7. Example of Failure of Industrial Facility Bus, Closed circuit breaker in Industrial Facility Bus fails to open on demand, and Switchgear on Bus at Industrial Facility load for Branch A.	60
Figure 6-8. Overcurrent by generator step-up transformer.	60
Figure 6-9. Breaker for generator (IE-TRANS-HES110)	
Figure 6-10. Transformer for generator (IE-TRANS-HES111).	
Figure 6-11. Breaker in industrial facility (IE-TRANS-HES1321) as an example for spurious	
failure of a circuit breaker	62
Figure 6-12. MSLB ET (IE-MSLB).	
Figure 6-13. MSLB ET with HES (IE-SLB-TOT).	
Figure 6-14. FT for total IE frequency for PWR MSLB	64
Figure 6-15. FT for Total Initiating Event frequency for MSLB with 100 MW industrial customer	65
Figure 6-16. FT for Total Initiating Event frequency for MSLB with 500 MW industrial customer	66
Figure 6-17. FT for Total Initiating Event frequency for MSLB with 1000 MW industrial customer	67
Figure 6-18. General Transient ET (IE-TRANS).	69

Figure 6-19. Initiating event for steam line break in the HES (IE-SLB-TOT).	70
Figure 6-20. Linkage rules for the IE-LSSB-HES ET	70
Figure 6-21. LSSB-HES flag editor.	70
Figure 6-22. RPS FT.	71
Figure 6-23. PCS FT.	72
Figure 6-24. PCSR FT.	72
Figure 8-1. Acceptance guidelines for CDF.	78
Figure 8-2. Acceptance guidelines for LERF.	79
Figure A-1. SGTR ET.	88
Figure A-2. ATWS ET	89
Figure A-3. Station blackout (SBO) ET	90
Figure A-4. SBO-1 ET.	91
Figure A-5. SBO-2 ET.	91
Figure A-6. SBO-3 ET.	92
Figure A-7. SBO-4 ET.	93
Figure A-8. Medium loss-of-coolant accident ET.	94
Figure B-1. General plant transient ET (IE-TRANS) Part 1, showing three truncated branches (i.e., branch A, B, and C)	96
Figure B-2. General plant transient ET (IE-TRANS) Part 2, revealing branch B and C	97
Figure B-3. General plant transient ET (IE-TRANS) Part 3, revealing branch A.	98
Figure B-4. One stuck-open relief valve ET (P1) Part 1, showing a truncated branch	99
Figure B-5. One stuck-open relief valve ET (P1) Part 2, revealing branch A	100
Figure B-6. One stuck-open relief valve ET (P1) Part 3, revealing branch B.	101
Figure B-7. Two or more stuck-open relief valves (P2).	102
Figure B-8. LOOP (plant-centered) ET (IE-LOOPPC).	103
Figure B-9. LOOP-1 ET (P1)	104
Figure B-10. LOOP-2 ET (P2)	105
Figure B-11. SBO ET.	106
Figure B-12. SBO-OP ET.	107
Figure B-13. SBO-ELAP ET.	108
Figure B-14. SBO-1 ET.	109
Figure B-15. ATWS ET.	109
Figure B-16. ATWS-1 ET	110

TABLES

Table 3-1. Overview of industrial facility requirements.	4
Table 4-1. Power ratings for proposed industrial customers and NPP	7
Table 4-2. Electrical demand of reference industrial customers.	
Table 5-1. Industrial facilities design options and assumptions.	25
Table 5-2. FMEA-derived potential failures from hazards and PRA ET assignment	28
Table 5-3. Methanol hazard summary.	30
Table 5-4. Refinery hazard summary	31
Table 5-5. Full-process kraft mill pulp hazards summary.	32
Table 5-8. Heat-transfer fluid properties.	36
Table 5-9. Flame speed Mach numbers (M _f) of BST method	40
Table 5-10. Distance to 1.0 psi for maximum hydrogen detonation scenarios for HTEF sizes as calculated using Bauwens-Dorofeev and TNT equivalent methods	40
Table 5-11. Syngas parameters.	47
Table 5-13. Extracted probabilities for high-voltage circuit breakers and transformers [61]	54
Table 5-14. Seismic fragility constants used for high-voltage relays, busbars, and switchgear	54
Table 5-15. Properties of control room air flow.	55
Table 7-1. Overall PWR core damage frequency results by HES modification.	73
Table 7-2. PWR Steam line break results by HES modification	73
Table 7-3. PWR Transient results by HES modification	73
Table 7-4. PWR Overall seismic results by HES modification	74
Table 7-5. Overall BWR core damage frequency results by HES modification	74
Table 7-6. BWR Steam line break results by HES modification	75
Table 7-7. BWR Transient results by HES modification	75
Table 7-8. BWR Overall seismic results by HES modification	75
Table C-1. Scoring criteria for FMEA ranking categories.	112
Table C-2. Risk Priority Number acronym descriptions	112
Table D-1. Nuclear power plant based FMEA results for SynGas.	114
Table D-2. Industrial customer based FMEA results for syngas	118
Table D-3. Public safety and perception based FMEA results for syngas.	121
Table E-1. Nuclear power plant based FMEA results for Methanol Synthesis Facility.	124
Table E-2. Industrial customer based FMEA results for Methanol Synthesis Facility	126
Table E-3. Public safety and perception based FMEA results for Methanol Synthesis Facility	128
Table E-4. Economy based FMEA results for Methanol Synthesis Facility.	129

Table F-1. Nuclear power plant based FMEA results for Petroleum Refinery Facility	. 132
Table F-2. Industrial customer based FMEA results for Petroleum Refinery Facility.	. 135
Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility	. 144
Table F-4. Economy based FMEA results for Petroleum Refinery Facility	. 155
Table G-1. Nuclear power plant based FMEA results for pulp and paper facility.	. 166
Table G-2. Industrial customer based FMEA results for pulp and paper facility	. 168
Table G-3. Public safety and perception based FMEA results for pulp and paper facility	. 169
Table G-4. Economy based FMEA results for pulp and paper facility	. 170
Table H-1. Lists of flammable and detonable products and feedstocks in methanol plant [2]	. 172
Table H-2. Lists of flammable and detonable products and feedstocks in Synthetic Fuel Plant	. 172
Table H-3. Lists of flammable and detonable products and feedstocks in Refinery Plant.	. 172
Table H-4. Lists of flammable and detonable products and feedstocks in Refinery Plant.	. 173
Table H-5. Lists of toxic products and feedstocks in Methanol Plant	. 174
Table H-6. Lists of toxic products and feedstocks in Synthetic Fuel Plant	. 174
Table H-7. Lists of toxic products and feedstocks in Refinery Plant.	. 175
Table I-1. Component Leak Frequencies	. 177



ACRONYMS

AFW auxiliary feedwater

ALOHA Areal Locations of Hazardous Atmospheres

ARIA Analysis, Research and Information on Accidents

ATE acute toxicity estimate

ATWS anticipated transient without scram

Bauwens - Dorofeev hydrogen jet leak detonation consequence methodology

BESS battery energy storage system

BLEVE boiling liquid expanding vapor explosion

BST Baker-Strehlow-Tang detonation consequence methodology

BWR boiling-water reactor
CCF common cause failures
CDF core damage frequency

CSB Chemical Safety and Hazard Investigation Board

CST condensate storage tanks
DBA design basis accidents

DDT deflagration to detonation transition

DI deionized

DOE U.S. Department of Energy
DRA deterministic risk assessment

EPA Environmental Protection Agency

ET event tree

FMEA failure modes and effects analysis

FPP fire protection plan FST finished short tons

FT fault tree

HES heat extraction system
HPI high-pressure injection

HTEF high-temperature electrolysis facility

HTF heat-transfer fluid IE initiating event

INL Idaho National Laboratory

LAR licensing amendment review

LERF large early release frequency

LOOP loss-of-offsite power
LPI low-pressure injection
LWR light-water reactor

LWRS Light Water Reactor Sustainability program

MSIV main steam isolation valves

NFPA National Fire Protection Association

NPP nuclear power plant

NRC U.S. Nuclear Regulatory Commission

OCA owner-controlled area

OSHA Occupational Safety and Health Administration

PAC Protective Action Criteria
PCS power conversion system
PRA probabilistic risk assessment
PWR pressurized-water reactor
RCP reactor coolant pump
RPN risk priority number

RPS reactor protection system
RWGS reverse water gas shift

S&L Sargent and Lundy company

SAPHIRE Systems Analysis Programs for Hands-on Integrated Reliability Evaluations

SBO station blackout

SME subject matter expert

SNL Sandia National Laboratories SOEC solid oxide electrolysis cells

SPAR-H Standardized Plant Analysis Risk – Human Reliability Analysis

SSC structures, systems, and components

STEL short-term exposure limit

synfuel Synthetic fuel (sustainable, non-fossil fuel-based fuels)
syngas synthesis gas, a mixture of carbon monoxide and hydrogen

TWA time-weighted average VCE vapor cloud explosions

Flexible Plant Operation and Generation

Hazards and Probabilistic Risk Assessments of a Light-Water Reactor Coupled with Industrial Facilities

1. INTRODUCTION

1.1 Nuclear-Supported Industrial Facilities

The U.S. electric power grid continues to evolve resulting in an emerging gap between the growth of non-dispatchable renewable energy generation and lagging clean energy storage that continues to contribute to the unproductive expansion of time-of-day excess clean energy generation. The overlapping impact of and competition between the dominant clean-generating sources (intermittent renewables and baseload nuclear power) exacerbates this challenge during daily supply-and-demand cycles.

A contributing factor is that both intermittent renewables and baseload nuclear power have inherent flexibility constraints in their operational models. Nuclear power has significant near-term potential to change its long-standing operational model by shifting generation output away from electrical generation when there is no additional grid demand for clean energy. During these times, nuclear could flexibly produce real-time usable or storable clean energy to decarbonizing functions across the power, industrial, and transportation sectors. Nuclear generated electrical and/or thermal energy can be used in many industrial processes beyond producing hydrogen. These industrial applications include decarbonization of oil refinery processes, producing methanol for synfuel production, and decarbonization of wood pulp and paper mills.

The U.S. Department of Energy's (DOE) support under the Light Water Reactor Sustainability (LWRS) Flexible Plant Operations and Generation (FPOG) Pathway at Idaho National Laboratory (INL) is accelerating key technology development in this area. The current LWRS research and development focus regarding implementation of integrated nuclear facilities with industrial facilities is being addressed through exploration of practical techno-economic analyses, use of pre-conceptual designs, pilot hydrogen projects, and development of the safety case through analyses supporting licensing success paths consistent with the NRC requirements (Figure 1-1).

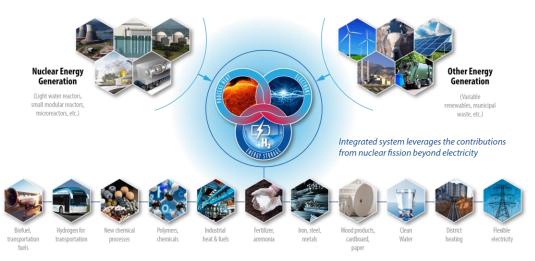


Figure 1-1. Nuclear can provide heat and electricity for many industrial processes.

This report has been developed as a key element of the FPOG Pathway program to support utility assessment of essential aspects for licensing approval of proposed modifications that facilitate thermal energy extraction from the nuclear power plant and provide electrical power to supply co-located industrial processes that provide for nuclear plant operational flexibility for economic value and decarbonization. Specifically, this report provides guidance for utilities for both hazards analysis and for the Probabilistic Risk Assessment (PRA) evaluation required as part of the modification process.

The following assembles hazard analyses that support these proposed modifications. The identified hazards provide input to the PRA model of the generic NPP and industrial facilities. The fragility of the NPP structures, systems, and components (SSCs) combined with deterministic consequence analysis were used to risk-inform the safe separation distance of the individual facilities from the NPP's SSCs. Procedures were investigated for setting the safe separation distance between the NPP and the industrial facility including the adherence to the NPP site fire protection plan and underlying code/licensing requirements. A deterministic approach was proposed for use to set the safe separation distance by using the criteria in U.S. Nuclear Regulatory Commission's (NRC) Regulation Guide 1.91 [3] even within the NPP's owner-controlled area (OCA) where it is not formally required. Modifications to the NPP and external hazards from each facility were added to existing PRA models, as appropriate.

1.2 Risk Assessment Roles in Safety and Licensing of Nuclear Power Plant Modifications

For the suggested change to the light-water reactor (LWR) design and operation to be approved, the NPP licensees must demonstrate that NPP safety will not be adversely affected. Both deterministic risk assessment (DRA) and probabilistic risk assessment (PRA) are used to risk-inform the decision for change acceptance by the NRC under licensee initiated 10 CFR 50.59, "Changes, Tests and Experiments" [4] evaluation or license amendment request (LAR). While the 10 CFR 50.59 evaluation does not specifically require a PRA, the PRA does provide numerical evidence of the effect of the proposed activities.

DRA sets criteria for safe siting distance between an NPP and an industrial facility. DRA also informs the inputs to the PRA. Hazards to the NPP presented by industrial processes are quantified through deterministic analyses. The hazard's effects versus distance are critical inputs for determining safe siting distance between the facilities.

Examples of DRA used for NPPs co-located with industrial facilities are blast overpressure, heat flux from fires, and concentrations of toxic chemical clouds.

PRA is a process by which risk is numerically estimated by computing the probabilities of what can go wrong and the consequences of those undesired events. The accident occurrence frequency to the probability of the NPP mitigating the accident without fuel damage are all quantified through PRA. The quantitative PRA results are compared to U.S. Code of Federal Regulations and NRC guidelines which determine if the design and operation are safe enough for approval or if changes need to be made to increase its safety.

An LWR PRA is broken into three levels, the first of which answers the risk-informed questions present in 10 CFR 50.59 [4]. These questions concentrate on the changes in IE frequency of design basis events caused by the proposed modifications. The Level 1 PRA also determines overall core damage frequency (CDF) and large early release frequency (LERF) which are metrics used in the risk-informed support of changes to licensing basis, NRC Regulation Guide 1.174 [5]. RG 1.174 can be used as further supporting information to back up decisions made in the 10 CFR 50.59 process, or as justification for an accepted LAR.

A Level 1 PRA estimates the frequency per year of CDF events. This is done using two types of logical structures—event trees (ETs) and fault trees (FTs). An ET represents the possible pathways that can occur due to an undesired outcome. The initial undesired event is called an initiating event (IE). After the IE, the ET uses FT model results to represent responding systems that prevent core damage. These FTs are the top events of the ET. The ET sequence of events results in end states indicative of the reactor state. The end state of interest here is core damage. All basic events of component or human action failures have associated probabilities of failure that are used in relation to one another as defined by the logic trees. The sum of the probabilities associated with all the sequences leading to the core damage end state represent the CDF.

Top-down methods are typically used to define IE frequencies by using data of recorded events to calculate the event frequency.

The probabilities of failure for FT top events are calculated using a bottom-up method. Bottom-up methods rely on knowing the exact system componentry and controls that are then translated into an FT. Typically, this is accomplished by referencing a system piping and instrumentation diagram and a list of operator actions, then identifying how each of those components and actions could fail in a way that leads to a failure event in the ET. The FTs are created and integrated into ETs by identifying within which IE the system failure would be used, either as an initiator itself or as a modification to one of the responding systems.

2. PROJECT SCOPE

The scope of this report is to assess hazards and consequences presented by representative industrial customer facilities located near an NPP. Hazards analysis is performed through the use of accidentology and failure modes and effects analysis (FMEA). A Level 1 PRA is modeled to assess the design basis IE frequency changes and the risk of core damage by quantifying the CDF associated with modifying the LWR to divert process steam and provide this thermal energy and a behind the meter electrical connection to the industrial customer. Other electrical powering methods of the industrial customer, including the NPP first connecting to the grid, are not potentially hazardous to the NPP, thus are not analyzed. Within the PRA, the industrial customer, and its electrical connection to the LWR are treated as both a potential internal and external event hazard upon the LWR. The IE frequencies associated with the addition of the proposed LWR heat extraction systems (HES) and the industrial customer facilities are compared against the guidelines set in 10 CFR 50.59, and the CDF and LERF that are calculated from the PRA are compared against the guidelines set in RG 1.174. Recommendations for the applicability of the results to this licensing path are given in this report.

The scope uses the representative industrial facilities (industrial customers) to perform a hazards analysis and facility siting analysis. The hazards analyses for these facilities provide quantitative input to the PRA of the NPP and deterministic quantifications used for safe separation distance siting analysis. The quantitative results from the deterministic analyses and the qualitative results from the FMEA are used to assess the risk to the local community and the economics of the NPP. Safe separation standoff distances between the NPP and the industrial customer are discussed, and regulations and codes are provided for determining them.

External events are assessed to determine if any of their effects on the industrial facility will affect the NPP.

Hazards from storage of industrial feedstock and products are also assessed, and a standoff distance calculation method is presented for assessment of acceptable risk to the NPP.

3. SPECIFICATIONS OF THE SUPPORTED INDUSTRIAL FACILITIES

Various industrial facilities were analyzed for integration with an NPP. The NPP can supply thermal and/or electrical energy to support the operation of the facility. The following sections describe these facilities and Table 3-1 shows a summary of the requirements of the various analyzed industrial facilities. The hydrogen High-Temperature Electrolysis Facility (HTEF) is analyzed at three different system capacities as shown in the table. The petroleum refinery thermal requirement includes both the heat from combustion (446 MWt) and steam (39 MWt). All other thermal power requirements are supplied by steam.

Table 3-1.	Overview	of industrial	facility	requirements.
I doic 5 I.		or maasara	Iuciii	requirements.

Process	Reference	Plant Size (/day)	Thermal Req. (MWt)	Electrical Req. (MWe)
II II: 1 T	[6]	54 MT	25	105 (100 MW _{nom})
H ₂ High-Temperature Electrolysis Facility		272 MT	105	500
Electrorysis Facility		544 MT ^a	205ª	1,000 ^a
Methanol Plant	[7]	1,340 MT	-19.7 ^b	24
Synthetic Fuel Production (Methanol Intermediary)	[7]	4,600 BBL	156	26
Synthetic Fuel Production (F-T)	[8]	4,405 BBL	73 (for HTEF)	437 (422 HTEF, 15 F-T)
Petroleum Refinery	[2]	100,000 BBL	485	28
Pulp and Paper Mill	[2]	1,095 finished short tons (FST)	156	25

a. Assumes two adjacent NPP unit-connected 500 MW_{nom} functioning in parallel as a single common facility with the same losses and margin.

3.1 High Temperature Electrolysis Hydrogen Facility

Production of hydrogen from electrolysis using solid oxide electrolysis cells (SOECs) or through proton exchange membrane processes is the most promising large-scale carbon-free method of producing hydrogen. The efficiency advantages of SOEC HTEFs make them more desirable when steam and electricity are both supplied. The NPP's ability to directly supply both electrical energy and thermal energy for process steam production without carbon emissions makes it ideal for this application. This report concentrates on SOEC HTEF designs. Preconceptual hydrogen HTEFs [1] specifications come from designs that are rated as 100 MW_{nom} and 500 MW_{nom} [6]. The 1000 MW_{nom} plant is assumed to be two adjacent 500 MW_{nom} facilities, therefore doubling the 500 MW_{nom} HTEF requirements. The electrical requirements match the nominal energy of the HTEFs, while the thermal requirements are 25, 105, and 205 MW_{tot} respectively.

3.2 Methanol Plant

Methanol is a valuable product because it has a wide variety of applications as a feedstock, such as for synthetic fuel production, as well as an end-use product. Currently, more than 85% of global methanol is synthesized from coal gasification and steam methane reforming [9]. These methods rely on coal or natural gas feedstocks to react with steam to form synthesis gas (syngas) which is a mixture of carbon monoxide and hydrogen. Syngas is then used to create methanol. Utilizing hydrogen generated via electrolysis and captured carbon dioxide process through selexol filtration, reduces the associated greenhouse gas emissions compared with the production of methanol generated with coal or natural gas-

b. 19.7 MWt generated, no thermal input is required.

based feedstocks [2]. This methodology utilizing electrolysis and carbon capture is analyzed in this report. The process and its requirements will be explained below.

The reverse water gas shift (RWGS) reaction generates the syngas which is used to produce the methanol in a series of fixed bed reactors [2]. The base plant size for reference utilized from [7] is 1,340 metric tons of methanol per day. This was determined from coupling with a 500 MW NPP. The thermal and electrical requirements are -19.7 MWt and 24 MWe, respectively, based off the methanol synthesis portion of an overall synfuel synthesis process. Values may be slightly higher than a standalone methanol synthesis facility. The thermal requirement is negative because the process generates 19.7 MWt. More about this process and the requirements can be found in [7].

3.3 Synthetic Fuel Production

Synthetic fuel (synfuel) is another valuable commodity. It is created using renewable feedstocks of hydrogen and carbon dioxide which significantly decrease the carbon usage compared to other fossil fuels such as petroleum and natural gas. The carbon dioxide used is captured from the atmosphere, neutralizing the net carbon associated with the process.

Two methods of synfuel production are considered in this analysis: production of synfuel with methanol as an intermediary product and the traditional Fischer-Tropsch process. Each of these processes share the same selexol-based carbon dioxide capture and RWGS reaction to create syngas from the carbon dioxide and hydrogen feeds, but the synthesis process differs. These two methods and their requirements are described in the following sub-sections.

These two methods also share the same final stages of fuel production as a traditional oil refinery. These are the separation and distillation, conversion through hydro-cracking, and tail gas utilization processes [7] as highlighted in Figure 3-1. For this reason, the hazards and accidentology for synfuel production are represented by analysis on refineries and are not repeated in the synfuel section. Combining the analyses of methanol production with the analyses of the common stages of the oil refinery result in an analysis of synfuel production plants.

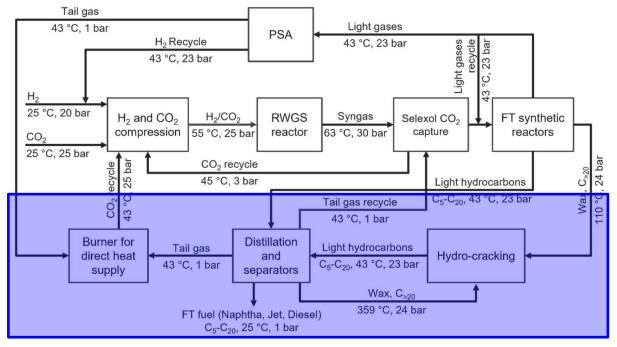


Figure 3-1. Synthetic fuel production process flow, highlighting the final stages that are similar to a traditional oil refinery [7].

3.3.1 Intermediary Methanol Product in Synthetic Fuel Plant

This method leverages methanol as an intermediary feedstock for synfuel production. Therefore, the process of synfuel production analyzed begins by using the methodology for methanol production explained above. Then, the methanol is converted into light olefins, primarily ethylene, propylene, and a minor amount of butene, then oligomerized into higher carbon length olefins. This results in a mixture of olefins that are mostly diesels. Finally, hydrogenation of the olefins are saturated into corresponding paraffins which are then separated within a fractionation unit to result in the following synfuels: naptha, jet fuel, and diesel.

The base plant size, for reference, produced 4,600 barrels of synfuel per day [7]. This was determined from coupling with a 500 MW NPP. The thermal and electrical requirements are 156 MWt and 26 MWe, respectively. Thermal power requirements can be delivered in the form of steam. More about this process and the requirements can be found in [7].

3.3.1 Fischer-Tropsch Process in Synthetic Fuel Plant

A reference Fischer-Tropsch (F-T) synfuel synthesis facility was adopted from a previous study [8]. The facility requires 255 MT/day H₂ and 1,580 MT/day CO₂ to produce 4,405 barrels of synfuel per day. The hydrogen feedstock is produced by an onsite HTEF with electricity consumption of 39.8 kWh/kg, resulting in an electricity demand from the NPP at 422 MWe. The HTEF requires thermal steam energy of 6.86 kWh/kg, resulting in a thermal demand of 73 MWt from the NPP. This hydrogen feedstock could be supplied from a 500 MW_{nom} HTEF. The F-T synthesis itself does not require NPP steam, but it requires a 15 MWe power supply. Therefore, the combined power drawn from the NPP is 437 MWe.

3.4 Petroleum Refinery

There are many products of a typical petroleum refinery and the decarbonization efforts are focused on the carbon dioxide produced when creating these products, not the reduction of carbon from when the products are used. Petroleum refineries are major contributors of carbon dioxide in industry. In a reference refinery plant using 100 thousand barrels per day [2], a total of 1.59 MT of carbon dioxide is generated. The reference refinery plant uses natural gas as a feedstock to provide heat and power to crack the crude oil and generate useful products such as asphalt, gasoline, diesel, and jet fuel. By integrating the refinery with an NPP, the usage of natural gas can be significantly reduced. The thermal requirement includes both the heat from combustion (446 MWt) and steam (39 MWt). The electrical requirement for a reference refinery is 28 MWe.

3.1 Pulp and Paper Mill

Paper in specific forms has a persisting and growing demand. Integrating with nuclear could be a way to ensure stability and growth over time. For this analysis, kraft pulping was chosen as the type of processing used because it currently makes up 80% of the total chemical pulping industry worldwide [10]. This pulping process includes dissolution of the wood chips liquor or chemical solutions to create a pulp product which can be processed into paper products. First, a white liquor solution is used to digest wood chips. Then the pulp is separated from the used cooking liquor and further refined via defibrating and bleaching stages if required to prepare the pulp for processing into paper products. The strength of the kraft process is the recycling of the liquors and heat through multiple stages. The spent cooking liquor is combined with pulp wash to create a black liquor. This is fired to recover heat for the pulping process. The inorganic chemicals of the black liquor are collected and dissolved in water to form a green liquor. Later, it is transferred to a causticizing tank to convert the solution back to white liquor for use in the digestion step again. For reference, the base plant size utilized from [2] is approximately 1,095 FST of paper product. The thermal and electrical requirements are 156 MWt and 25 MWe, respectively. Thermal

power requirements can be delivered in the form of steam. More about this process and the requirements can be found in [2].

4. NUCLEAR POWER PLANT MODIFICATIONS FOR AN INDUSTRIAL CUSTOMER

There are two NPP system modifications proposed. The first is adding the HES to extract thermal power and provide it to the industrial customer. The second is adding components to the switchyard necessary to provide direct electrical coupling to the industrial customer. Not all industrial customers will choose to use both thermal and electrical energy sources. Hydrogen HTEFs will use both, but it is at other industrial facility's discretion to choose what energy sources delivery to use from the NPP. The specifications below assume that both energy sources are required from the NPP.

4.1 Nuclear Power Plant with Heat Extraction System

Estimated power ratings required for the industrial processes investigated in this report are listed in Table 3-1. The heat extraction systems (HES) requirements for the differing processes and their percentage of steam extraction from a reference pressurized-water reactor (PWR) NPP are listed in Table 4-1.

Required Thermal Power	Reference Industrial Customer	Reference Nuclear Power Plant ¹			
MW_t		Full MW _t	Full MW _e	% Steam Extraction (MWt HTEF/ MWt NPP)	HES Steam Tap Location
156	Synthetic Fuel Pulp and Paper Mill	3650	1237	4.27%	After high- pressure turbine
485	Petroleum Refinery	3650	1237	13.29%	Main Steam before high- pressure turbine

Table 4-1. Power ratings for proposed industrial customers and NPP.

Note that although this report evaluates risks and design configuration associated with heat extraction from a Boiling Water Reactor (BWR) in similar fashion to the PWR design, this type of steam extraction design has pre-conceptually been evaluated and in detailed design practice is expected to involve series reboilers with an intermediate pressurized water loop between the NPP and HTEF or industrial user to provide additional barriers against transfer of radioactive contaminants downstream of the reboiler(s). Further, it is recognized that industrial steam users often require superheated process steam and that all NPP steam is effectively saturated. Although solutions to increase LWR NPP saturated steam to superheated conditions are currently planned to be pre-conceptually developed through research, such modification elements are excluded from the scope of this study and are not expected to significantly impact the risk results provided.

None of these designs are meant to represent a site specific modification and this should be considered when applying site specific analyses based on the outline of this report.

7

¹ MWt and MWe values are based on a generic Westinghouse 4-loop PWR plant but are assumed reasonably representative of all PWR plants at nominal operating conditions for the purposes of estimating thermal and electrical performance/output...

4.1.1 Low Level Heat Extraction System Design (≤ 100 MWt)

The HES for industrial customers with thermal requirements up to 100 MW_t is shown in Figure 4-1. The modifications required of the NPP are a steam tap prior to the HP turbine, a control valve system controlled by the NPP, steam piping leading to a qualified building or structure adjacent to the turbine building, steam connection to a reboiler fed by deionized (DI) water from the industrial customer, steam piping leading to the industrial customer, and DI water piping returning from the industrial customer [6]. The reboiler is placed the qualified building outside of the turbine building for space consideration, isolation for maintenance, and to protect the turbine building equipment.

The modifications required of the NPP for up to a $100~\text{MW}_t$ industrial customer are a steam tap after the HP turbine, a control valve system controlled by the NPP, steam piping leading to a building adjacent to the turbine building, steam connection to a reboiler fed by DI water from the industrial customer, steam piping leading to the industrial customer, and DI water piping returning from the industrial customer [6].

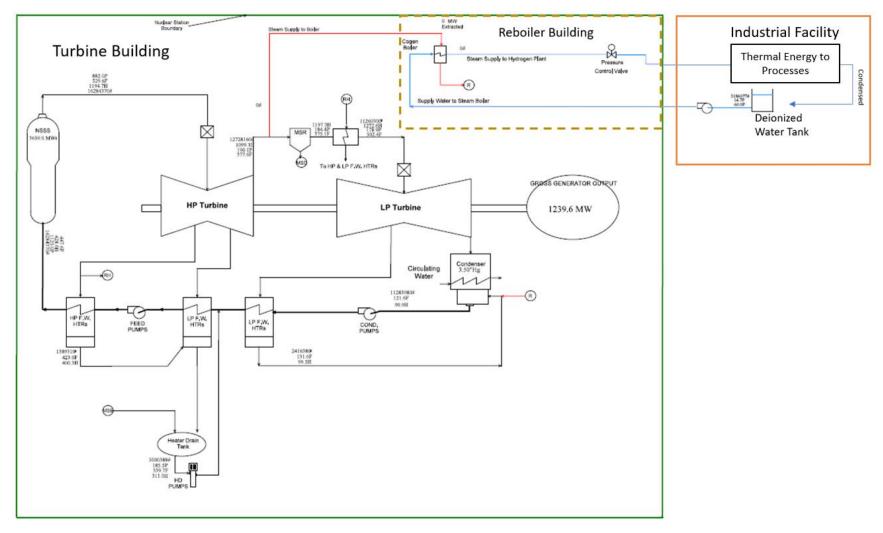


Figure 4-1. Model drawing of low-level HES (<200 MW_t).

The PRA requires a specification of the components added as a part of the modification of the NPP that affect the safety of the power plant. The diagram of a steam extraction line downstream from the HP turbine leading to the reboiler is shown in Figure 4-2 [6]. The diameter of the piping for the 100 MW_{nom} industrial customer is 10 in., 240 ft in length. This results in a maximum steam velocity of ~120 ft/sec. P1, P2, P3, and P5 are each 10-ft long with two 90-degree elbows. P4 is 200-ft long. A design pressure of 250 psig and design temperature of 400°F is assumed. J1 is the tap from the main steam, J2 and J5 are gate valves that are normally open in HES operation. J3 is a flow control valve with a constant pressure drop of 20 psig, assumed to have no flow-stopping capability. J4 is a stop check 90-degrees globe valve. J6 is the inlet to the reboiler. The pipe's insulation is assumed 4.5-in.-thick Calcium Silicate. The piping is located inside the turbine building, with an assumed indoor temperature of 70°F and air velocity of 0.1 ft/sec [6].

Since a failure in steam extraction lines up to, and including, the reboilers will affect the main steam line of the NPP and lead to an increased risk to the NPP, an FT for the line is developed, as shown in

Figure 6-15.

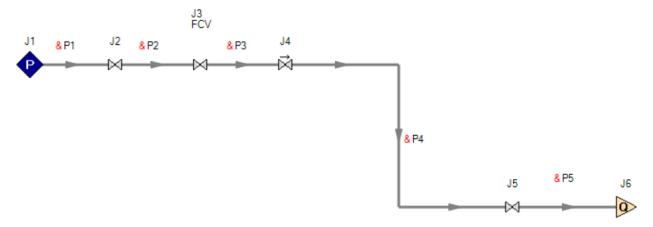


Figure 4-2. \leq 100 MW_t industrial customer diagram of steam extraction piping to the reboiler [6].

The reboiler required for heat transfer to the industrial facility is located within the NPP protected area in a reboiler building adjacent to the turbine building. None of the reference industrial customers process requirements will be satisfied by the mass flow rate associated with this lower-level of heat extraction, but hydrogen production facilities up to around 400 MW $_{nom}$ combined power requirements can use this HES design [1]. Since the amount of extracted steam (up to 2.75%) is much lower than the typical upset capacity of most NPP designs (approximately 30%), this extraction process will not affect normal plant operation. This design is for extracting low levels of steam. As an example, if 25 MW $_{t}$ power is required, 20 MWt is used to generate hydrogen while the remaining 5 MW $_{t}$ is a margin to cover various thermal losses.

4.1.2 Mid-Level Heat Extraction System Design (100 MWt to ≤ 200 MWt)

The HES for an industrial customer requiring a range between 100 MW_t and 200 MW_t is shown in Figure 4-3. The modifications required of the NPP are two steam taps after the HP turbine, a control valve system controlled by the NPP, steam piping leading to a building adjacent to the turbine building, steam connection to two reboilers fed by DI water from the industrial customer, with steam piping leading to the industrial customer, and DI water piping returning from the industrial customer [6]. The reboilers are placed in their own building outside of the turbine building for space consideration, isolation for maintenance, and to protect the turbine building equipment.

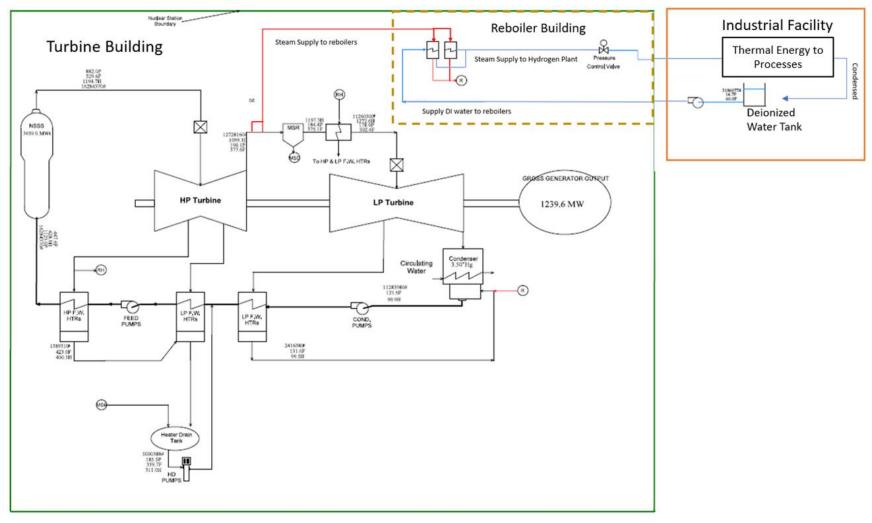


Figure 4-3. Model drawing of mid-level HES (> 100 MW_t to $\leq 200 \text{ MW}_t$) industrial facility requirement.

The PRA requires a specification of the components added as a part of the modification of the NPP that affect the safety of the power plant. The diagram of a steam extraction line downstream from the HP turbine leading to the reboiler is shown in Figure 4-4. The diameter of the piping header (P5) for the MidLevel HES up to the reboilers is 20 in., 200-ft in length with 14-in. branches, from two taps after the HP turbine and splitting again to two reboilers, a total of 60-ft for each train. This results in a maximum steam velocity of 150-ft/sec at an assumed mass flow rate of approximately 355,000 lbm/hr. J1 and J21 are taps from the cold reheat discharge from the HP turbine, J2, J7, J22, and J27 are gate valves that are normally open in HES operation. J3 and J23 are flow control valves with a constant pressure drop of 20-psig, assumed to have no flow-stopping capability. J4 and J24 are stop check 90-degrees globe valves. J8 and J28 are the inlets to the reboilers.

A failure in the steam extraction system up to and including the reboilers will affect the main steam line of the NPP and lead to an increased risk to the NPP. An FT for the line is developed as shown in Figure 6-16.

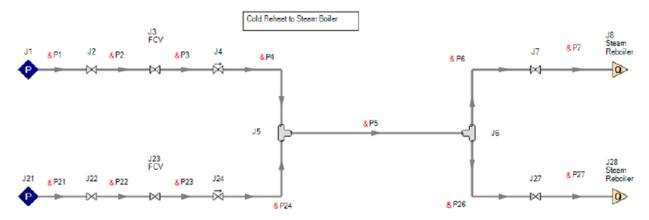


Figure 4-4. 100 MW_t to \leq 200 MW_t industrial customer diagram of steam extraction piping to the reboiler [6].

The reboilers required for heat transfer to the industrial facility are located within the NPP protected area in a reboiler building adjacent to the turbine building (Figure 4-3). Two of the reference industrial customers (synthetic fuel plant and pulp and paper mill) require 156 MW $_t$ extraction, which lies within this mid-level range. Since the amount of extracted steam at 156 MW $_t$ (4.27%) (Table 4-1), is much lower than the typical upset capacity of most NPP designs (approximately 30%), this extraction process will not affect normal plant operation. The reference design is for extracting 161 MW $_t$ of steam. Out of this 161 MW $_t$ power, 156 MW $_t$ is used to at the industrial facility while the remaining 5 MW $_t$ is a margin to cover various thermal losses.

4.1.3 High-Level Heat Extraction System Design (> 200 MWt)

It is important to note that unlike the low and mid-range HES designs, systems requiring greater than 200 MW_t were not evaluated by Sargent & Lundy architectural engineers (S&L). The design described here uses guidance of the general layout from S&L, but the pipe sizing and lengths were specified through engineering judgment by the INL PRA team. Originally, before the S&L studies, a 15% steam extraction case was modeled in INL/EXT-21-63225, "Evaluation of Different Levels of Electric and Thermal Power Dispatch Using a Full-Scope PWR Simulator," [11] where a 20-in. steam pipe was used. The required 5.62% steam extraction for the high range industrial customer is much less than the 15% steam extraction model in Reference [11]. The assumption is made that dividing the cross-sectional area of the 20-in.-diameter pipe in half is a conservative estimate to determine the size of steam pipe required. The result of

this is a 14-in. diameter pipe. Pipe length before the branches to the three reboilers was assumed to be the low-level HES length with an additional 40 ft added because of the increased distance to the main steam line. The pipe sizes and lengths of the reboiler branches were assumed to be 12 in., which is slightly less than the mid-level HES because of the higher energy of the main steam.

The high-level HES is shown in Figure 4-5. The modifications required of the NPP are a steam tap prior to the HP turbine at full main steam system pressure and temperature, a control valve system controlled by the NPP, steam piping leading to a building adjacent to the turbine building, steam connection to three reboilers fed by DI water from the industrial customer, steam piping leading to the industrial customer, and DI water piping returning from the industrial customer [6]. The three reboilers are placed in their own building outside of the turbine building for space consideration, isolation for maintenance, and to protect the turbine building equipment.

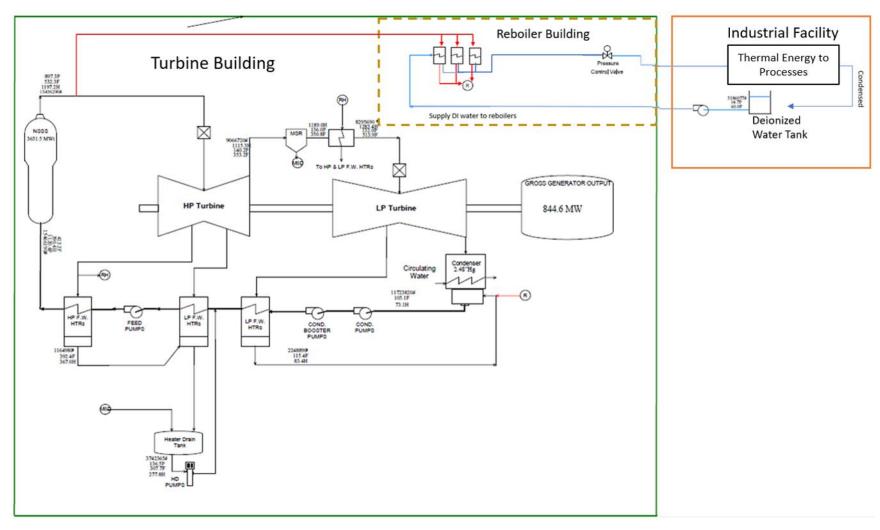


Figure 4-5. Model drawing of high-level HES (>200 MW_t).

The PRA requires a specification of the components added as a part of modifications to the NPP that affects the safety of the plant. Shown in Figure 4-6 is the diagram of a steam extraction line downstream from the main steam tap that leads to the three reboilers. The diameter of the piping for the previously evaluated Mid-Level industrial customer is 14 in., 240-ft in length, from the main steam tap to the three reboilers, which are 12-in. pipe branches of a total of 60-ft for each train. This results in a maximum steam velocity of ~150-ft/sec. J1 is a tap from the main steam line prior to the HP turbine, J2, J6, J16, and J26 are gate valves that are normally open in HES operation. J3 is a flow control valve with a constant pressure drop of 20-psig, assumed to have no flow stopping capability. J4 is a stop check 90-degrees globe valve. J7, J17, and J27 are the inlets to the reboilers.

Since a failure in any steam extraction lines up to, and including the reboilers will affect the main steam line of the NPP and lead to an increased risk to the NPP, a FT for the line is developed as shown in Figure 6-17.

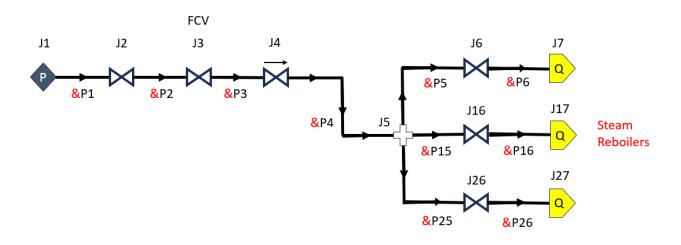


Figure 4-6. > 200 MW_t industrial customer diagram of steam extraction piping to the reboilers.

The reboilers required for heat transfer to high-level thermal requirements industrial customers is located within the NPP site in a reboiler building adjacent to the turbine building (Figure 4-5). The steam extraction operation for the high-level thermal extraction is from main steam and it operates like an auxiliary steam system that has been used in some existing NPPs. The reference industrial customer for this level of thermal extraction is a petroleum refinery. Since the amount of extracted steam (13.29%) (Table 4-1) is lower than the typical upset capacity of most NPP designs (approximately 30%), this extraction process is not expected to affect normal plant operation other than the different control schemes required to manipulate main steam diversion. The reference design is for extraction of 490 MW_t of steam. Out of this 490 MW_t power, 485 MW_t is used in the refinery, while the remaining 5 MW_t is a margin to cover various thermal losses.

4.2 Direct Electrical Connection

The example provided here is behind the meter that taps from the generator step-up (GSU) transformer. This example is one of several options. For instance, a plant specific evaluation may establish that a connection to a unit GSU transformer may not be feasible and or a multi-unit site may need to power a centrally located facility (e.g., one HTEF facility at a two-unit plant) necessitating a connection to the plant switchyard. While this GSU example provides a roadmap for evaluating these

types of modifications, this again is an area where the plant specific analysis will dictate the configuration.

Figure 4-8 and Figure 4-9 show the electrical connection to the industrial customer is assumed to run from a tap just outside of the NPP main GSU transformer to the switchgear at the industrial customer. The transmission line distance is determined by the safe standoff distance from the hazards analysis, high-voltage (typically 345 – 525 kV) line with protection at each end, a circuit breaker with manual disconnect switches on each side, and primary and backup relays. The first circuit breaker downstream of the tap point also electrically separates the transmission from the NPP switchyard breaker alignment. As stated in Section 4.3.5 of Reference [6], "The new H2 power line has no effect on the switchyard voltage, breaker alignment, generator automatic voltage generator loading, or the status of offsite power voltage regulating devices." This eliminates the impact of the transmission line on NPP safety systems that rely on offsite power.

A three winding step-down transformer steps the line voltage down to the 13.8-kV medium voltage required at the switchgear for the industrial customer. The switchgear at the industrial customer is interpreted as drawn, with a circuit breaker-protected bus with four inputs on each winding. The transformers and generator circuit breaker (GCB) also have primary and backup relays. Control panels and power for the relays before the transmission line are within the NPP switchyard. Then there is a transmission line run over the determined safe separation distance to the industrial customer (Figure 4-9), where protective circuits receive the power from the NPP. Should these protections fail in an overcurrent event due to loads at the medium voltage switchgear or either of the transformers, the resulting overcurrent at the generator could cause a turbine upset transient event at the NPP. This failure model is detailed in Section 6.1.

Alternatively, if the line were to experience a faulted trip, simulations conducted in Section 4.1.3.8. of [6] show that a fault on the three-phase line must be cleared within 0.2 seconds or else it would destabilize the generator and cause a transient at the NPP. The designed load of the electrolysis process and total electrical demand of the entire HTEF are detailed in Table 4-2. The loads detailed for the 1000 MW_{nom} HTEF are assumed to be a linear scaling (double) of the 500 MW_{nom} HTEF since we assume that the 1000 MW_{nom} HTEF is two adjacent 500 MW_{nom} HTEFs as detailed in Section 3.

Apparent power (S), colloquially known as the total electrical demand that needs to be delivered from the power plant, is the complex sum of real power (P) and reactive power (Q) [12]. This relationship is illustrated by the power triangle diagram shown in Figure 4-7, where S [VA] = P[W] + jQ[VAr]. P is the power that the industrial facility needs to perform its function, while Q is the power required to overcome the net reactance from power cables and transformers in the behind-the-meter AC transmission line.

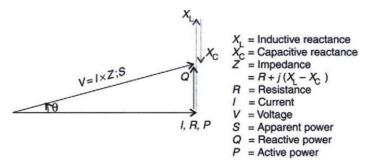


Figure 4-7. Power triangle diagram [12]

The existence of reactance within the network leads to a phase shift between the voltage and current phasors at the load, with a phase angle denoted by theta (θ). Here, the voltage phasor serves as the reference point, and the current phasor is described as "lagging" when considering a counterclockwise rotation of the phase. The power factor (PF) is represented by the cosine of this phase angle ($\cos \theta$), and

in conjunction with Figure 4-7, is therefore formulated by Equation (1). A theoretical ideal of the power factor is 1 (i.e., no phase angle due to no network reactance). In reality, the power factor may be around 0.9. The longer the cables and the more transformers are in the transmission line, the higher inductive reactance (X_L) is, and the power factor decreases. It physically means that there are more electrical losses. However, the power factor can be improved by adding capacitance to an inductive network to increase capacitive reactance (X_C) in what is known as power factor correction.

$$PF = \cos \phi = \frac{P}{\sqrt{3} \times V \times I} = \frac{P}{S} \tag{1}$$

The 100 MW_{nom} HTEF requires 105 MWe active power to perform electrolysis. An additional 10% active power is assumed for plant auxiliaries and ancillary loads, with another 10% margin to account for fluctuations. The resulting active power (P) becomes 120% of 105 MWe which is 126 MWe. The power factor (PF) of the transmission line was designed to be 0.92 (i.e., a phase angle of 23°) by utilizing capacitor banks to provide power factor correction and to compensate for transformer reactive power losses (Q) [6]. Therefore, the apparent power (S) for the 100 MW_{nom} HTEF comes down to $126/0.92 \approx 140$ MVA. Similar assumptions were applied to the 500 MW_{nom} HTEF, excluding the 10% active power margin since minor fluctuations are non-issues in such a high active power rating, to come to the apparent power rating (S) of $550/0.92 \approx 600$ MVA. Since the 1,000 MW_{nom} HTEF is a dual 500 MW_{nom} HTEFs, its apparent power is 1,200 MVA. By applying the same set of assumptions to other industrial facilities, their apparent power is between 30 to 40 MVA. These values are listed in Table 4-2.

Table 4-2. Electrical demand of reference industrial customers.

Reference Industrial Customer	Electrical Load (Active Power) [MW _e]	Total Electrical Demand (Apparent Power) [MVA]
Hydrogen HTEF [1]	100	140
	500	600
	1,000	1,200
Synthetic Fuel Plant [7]	26	35
Wood Pulp & Paper Mill [2]	25	35
Petroleum Refinery [2]	28	40

Considering these industrial customer design features and most recent data, no additional over-current protection is recommended.

Overcurrent Protection Beyond the Reference Industrial Customers: If the NPP is tasked to provide larger behind the meter loads, for instance a direct connection to a data center, it may be advisable to seek out further overcurrent protection. Initial research was performed in this area. First, a dump load or battery energy storage system (BESS) was considered for load shedding. Current literature on dump loads and BESSs shows applications for microgrids, renewable energy, and other systems that are smaller in electrical demand by an order of magnitude or greater [13]. Unfortunately, there is no indication that there is history or consideration of such protection even for the MV/MW level systems such as the HTEFs considered in this report. Feedback from subject matter experts (SMEs) and industry also support this observation [14]. HTEF SMEs at INL have explained that there are very few SOEC systems at industrial scale worldwide (the few being in Europe), so it is difficult to determine the nature of any overcurrent situations, and the protections required at that scale. Industry and other renewable energy SMEs have so far only referenced smaller magnitude power demand systems or have not indicated wide discussion or concern for dump load, BESS, or other load shedding protections for MV/MW level systems.

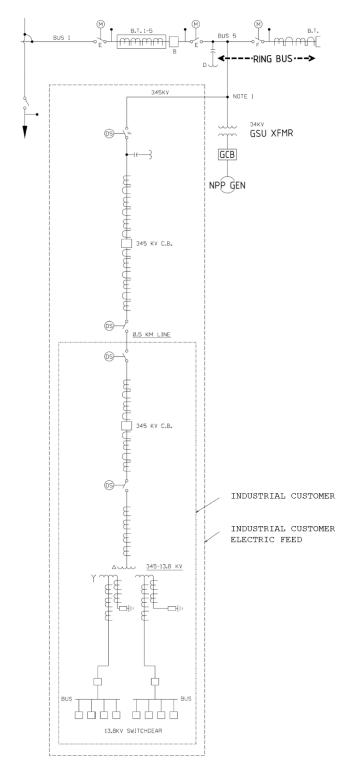


Figure 4-8. Transmission line and portion of ring bus switchyard arrangement at NPP [6].

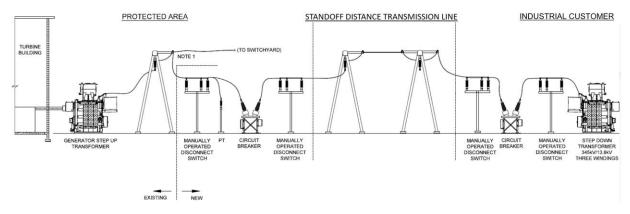


Figure 4-9. Behind-the-meter physical layout of electrical feeder [6].

5. HAZARDS ANALYSIS OF A NUCLEAR POWER PLANT SUPPLYING ENERGY TO AN INDUSTRIAL FACILITY

The hazards associated with co-locating an industrial process next to an NPP were researched through accidentology studies of historical industrial accident databases, identification of products and feedstocks and their properties, and through interviews and FMEA input from SMEs, utility engineers, S&L architectural engineers, and hydrogen experts at Sandia National Laboratories (SNL). Proposed design drawings and options of the conceptual HES were reviewed and evaluated in a system-level FMEA.

5.1 Accidentology

Industrial accidents are reported and recorded by safety agencies around the world. The study of trends and frequency of these accidents is called accidentology. Accidentology identifies what has happened at these facilities. This is beneficial in determining accident frequencies and consequences. The hazards identification and consequence quantification process continue with assessing the properties of the hazards, regardless of whether the hazard has manifested into an accident at the industrial site. We reviewed databases from the U.S. and internationally for each type of industrial customer considered in this report.

5.1.1 Hydrogen Electrolysis

We have included the hydrogen electrolysis accidentology even though hydrogen electrolysis is not one of the direct reference industrial customer facilities studied for this report. The reason is that production and use of hydrogen is required for all the reference facilities.

Worldwide incidents involving hydrogen are reported in the hydrogen incident and accident database (HIAD) maintained by the European Commission [15]. As of February 11, 2024, there are 755 events recorded in the database, 162 of which happened in the United States. The statistics of all incidents are shown in Figure 5-1. The top three causes of these incidents are management factors, material/manufacturing error, and human factors. Wen et.al. explained these factors as follows [16].

- Management factors: poor management planning causing overstressed workforce; failure to learn from previous incidents; lack of clear definition of responsibilities; poor management of health and safety, etc.
- Human factors: low competency levels; fatigue; disheartened staffs; medical problems, etc.
- Material/manufacturing error: Components malfunction due to material failures or manufacturing errors.

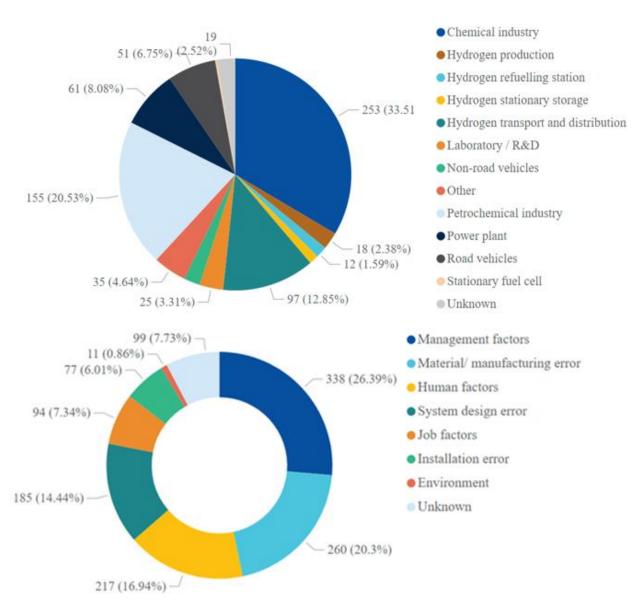


Figure 5-1. Hydrogen incident statistics per application type (top) and their causes (bottom) [15].

The "soft factors" include management factors, human factors, and job factors, which contribute to half of the incidents. Wen et al. highlighted that most of the incidents under this category were caused by a lack of regular/appropriate maintenance and inspection, and lack of attention for safety devices during maintenance and inspections such as fittings, gaskets, flanges, and valves. Lack of adequate staff training exacerbates these issues. Management factors contribute to these incidents through the lack of safety supervision during certain repair work, lack of adequate procedures, and lack of clear guidance about lifetime of critical components. These areas can be improved through regulations and establishing a good safety culture. Meanwhile, for the technical aspects, hydrogen gas itself easily dissipates into the atmosphere when leaked because it is lighter than air. Therefore, a hydrogen cloud detonation event creating a large overpressure is unlikely if confinement safety protocols are followed as prescribed by the National Fire Protection Association (NFPA) standard NFPA-2 [17].

Approximately 74% of the hydrogen incidents caused fire and/or detonations. The incident consequences per specific application supply chain stage are plotted in Figure 5-2. As the figure shows, about 30% of accidents happened when hydrogen was used as a process gas, mostly in the petrochemical industry. For example, two separate accidents occurred in 2022 that involved fire due to hydrogen leakage from a hydrogen compressor of a reforming unit at a refinery. In both cases, the hydrogen emergency flow cutoff was activated to stop the hydrogen leak, and a protective combustion was carried out following the safety protocol to decompress the unit.

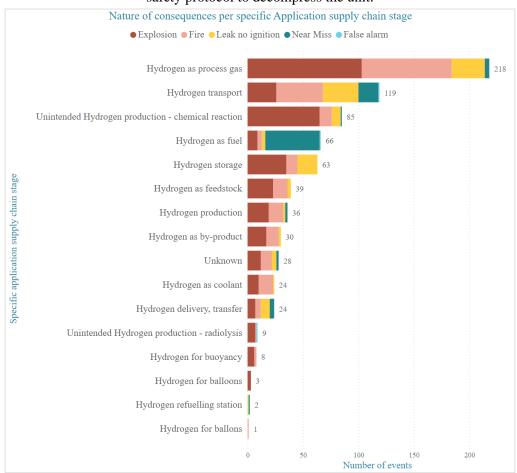


Figure 5-2. Hydrogen incident consequences per specific application supply chain stage [15].

Of all the recorded incidents, about 5% of cases happened during the hydrogen production process. The causes of these incidents are plotted in Figure 5-3. The top causes are similar to the overall recorded causes in Figure 5-1, except those contributions from human factors decreased and they are replaced by contributions from system design errors. This is because hydrogen production is less dependent on human actions compared to other activities (e.g., hydrogen transport and hydrogen refueling). An example of system design error is the explosion of three hydrogen buffer tanks at an experimental facility in South Korea in 2019. These tanks were receiving hydrogen produced from electrolyzers powered by solar panels. An investigation revealed the root cause was a static spark that ignited oxygen levels above 6% in the hydrogen tank, which is the minimum for an explosion. This unacceptable oxygen level was caused by the electrolyzer being run below its required power level. This minimum power level was required to operate the asbestos separation membrane, which drew in half of the electrical power supplied. Unfortunately, the electrolyzer often received subpar power because the solar panels' output fluctuated

with sunlight exposure. A contributing cause was that the system lacked oxygen removal devices and anti-static systems.

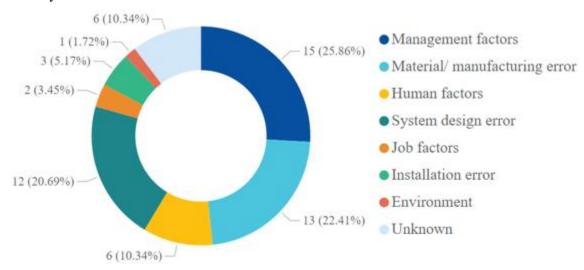


Figure 5-3. Accident causes during hydrogen production [15].

5.1.2 Methanol Plant Accidentology

Methanol accidentology overall shows that many accidents occur because of lack of inspection. When operators are unaware of the concentration or presence of methanol in the system or environment, adding heat to the system or environment can result in a rapid change in conditions. The consequences of known accidents have resulted in equipment rupture and burning, death, and injury.

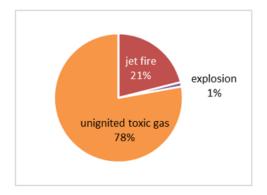
The overall frequency of methanol-related accidents includes a variety of situations. The U.S. Department of Labor Occupational Safety and Health Administration (OSHA) records 26 accidents related to methanol, with 12 accidents resulting in fatalities [18]. From the Analysis, Research, and Information on Accidents (ARIA) database [19], there were 149 records of accidents related to methanol. A record of an accident in the ARIA database highlights that there can be multiple ignition sources. To remove palladium residue from a chemical reactor, technicians cleaned it with boiling methanol. After cleaning, the opening of the reactor was not immediately closed. During this time, the residual methanol vapors from the opening were ignited. The most likely cause reported was from a palladium, methanol, and oxygen reaction or, less likely, but still possibly, from an electrostatic discharge from a nearby document console. Although palladium is not a catalyst material used in methanol synthesis from syngas, it is important to note that mixtures of methanol with other streams may increase likelihood of ignition.

In the U.S. Chemical Safety and Hazard Investigation Board (CSB) database, there is less direct searchability and accounting for methanol related accidents [20]. No direct results were found for the synthesis of methanol from syngas. Some results were found where methanol was used in other synthesis reactions, used as a cleaner for other chemical plants, or used in a mixture for other chemical reactions. The CSB notably reported on an investigation of the Bethune Point Wastewater Treatment Plant accident and a summary of other flammable gas accidents. The wastewater treatment plant had a continuous feed of methanol and a 10,000-gallon storage tank. As an operator was using a cutting torch to remove the metal roof directly above the methanol tank, vapors coming from the tank vent were accidentally ignited. This also led to the flame flashing back into the storage tank that resulted in an explosion inside the tank that created multiple methanol piping failures and a large fire that engulfed the tank and workers. Two workers died and another was severely burned. The results of the investigation reported that the cause of this accident was due to a lack of inspection and maintenance of the flame arrestor. The vent through the flame arrestor was constantly open and therefore always discharging methanol vapors due to the corrosion

of the arrestor by the methanol. In this degraded state, it did not prevent fire outside of the tank from igniting the tank's contents. The CSB also published a lesson learned on preventing deaths during hot work in and around flammable gas tanks based on explosion and fire accidents similar to the one at the Bethune Point Wastewater Treatment Plant [21]. The lesson they listed as the most important was to analyze the hazards and to monitor for combustible gas as a sign of a potentially flammable atmosphere.

5.1.3 Syngas Production Accidentology

Currently, there is no history of accidents involving syngas. Stolecka and Rusin [8] analyzed possible hazards related to syngas by developing an ET to track various consequences following damage to a syngas pipeline. The probabilistic consequences are shown on the left-hand side of Figure 5-4. The most likely consequence is that syngas is dispersed without ignition. This is because the flammable elements are diluted with non-flammables such as CO₂ and H₂O, thereby increasing the mixture's lower flammability limit and flash point. They also analyzed the radius of consequences for a reference coal and biomass plant. The maximum distances of those consequences are shown on the right-hand side of Figure 5-4. For that reference plant, the safe distance perimeter may be set against the jet fire hazard of 126 meters, which also accommodates explosive and toxicity risks.



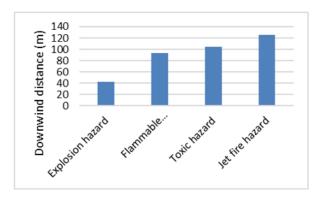
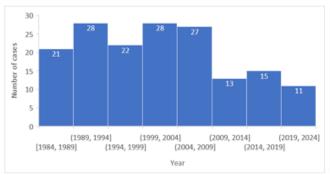


Figure 5-4. Probabilistic consequences of syngas release.

5.1.4 Oil Refinery Accidentology

The statistics from OSHA of 165 accidents in oil refineries from 1984 to 2024 are shown in Figure 5-5, 58 of which caused fatalities [18]. Among them, the Texas city disaster was believed to be the most catastrophic refinery accident in history, killing a total of 581 people including dockworkers [18] [19], residents, and sailors, and more than 5,000 people were injured. More than 150 miles of the areas from the ignition points were impacted. The root cause of this accident is still unknown, but a welder's torch was suspected to be the source of the ignition point. The Texas city disaster and three other events resulted in changes to the regulations. The root causes of the fires and explosions documented for these events were the release of flammable chemicals due to (1) rupture of the pipe or tank, (2) inadequate training of the workers, and (3) improper installation of the equipment. Pipe or tank rupture can result from the long-term degradation of the materials from the corrosive liquids such as hydrogen sulfide or the shock rupture due to an overpressure event in the systems. While the degradation effects on the piping or tank can be detected and repaired during the maintenance period, the shock rupture would be challenging to predict even if a pressure-monitoring system is installed. Safety features should be improved to mitigate the potential fires if those pipes or tanks containing flammable liquid or gas fail. For inadequate training and installations, a more rigorous preventive maintenance and scheduled training are required to prevent accidents.



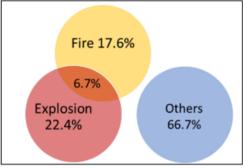


Figure 5-5. Historical statistics of accidents in oil refineries: over time (left) and consequences (right).

As for the cause of accidents, detailed refinery accidents reported by the CSB [20] show that most accidents are caused by the confinement of vapors, piping and heat exchanger failures, and inadequate procedures or human actions. There were two accidents associated with the confinement of vapors that happened when a buildup of flammable vapors ignited with an ignition source. For example, an overflowed flammable vapor cloud flowing down to the ground ignited with an idling diesel pickup truck present during the start-up of a raffinate splitter tower at the BP Texas City Refinery in 2005 [22].

Most accidents related to refineries in the U.S. CSB database were caused by piping and heat exchanger failures, and inadequate procedures and human actions. There were six accidents associated with piping and heat exchanger failures where the flammable liquid leaking from the failed structure ignited or exploded based on the pressure conditions. Piping and heat exchanger failures can be prevented by a preventive maintenance program and can help detect the precursor (e.g., crack initiation and crack propagation) of the accidents. Methodologies for piping reliability analysis that considered the underlying failure mechanisms and maintenance activities were reviewed in Reference [23].

Meanwhile, there were six accidents associated with inadequate procedures or human actions. In this type of accident, workers failed to operate a critical system or component because they inadequately followed the operation procedure or there was miscommunication between the workers that led to a catastrophic consequence. Inadequate procedures and human actions can be mitigated by regular training and licensing requirements for the operators and updating operating procedures to provide more clarity and guidance.

To prevent fire due to high flammability of feedstocks and products, fire protection programs should be enhanced and tested in refinery plants. The impacts of an overpressure event can be mitigated by adjusting the distances between the industrial applications and NPPs [1]. However, longer distances between a refinery plant and the NPP will increase the cost of thermal delivery systems due to the cost of piping material and energy lost during transport. INL is performing ongoing research to optimize the heat delivery and transportation costs between the NPPs and industrial applications [24].

5.1.5 Pulp and Paper Mill Accidentology

Common accidents for the pulp and paper industry include chemical exposure and burns, fires, explosions, water contamination, and mechanical accidents, (e.g., falls or hands/fingers caught in machinery). The ARIA database yielded ample results, including nine accidents uncovered when searching "pulp mill," and 36 when searching "paper mill" [19]. Only one accident extended outside the mill perimeter. Accidents listed in the OSHA database on pulp and paper mills are largely mechanical accidents pertaining to operating equipment and the handling or moving of product [18]. Pulp and paper mill accidentology revealed many common accidents stemming from various causes. Accidents listed in the CSB database included a significant explosion and an incident of H₂S toxicity, both of which involved fatalities, though neither extended beyond the mill boundary [20].

Based on accidentology studies of pulp and paper mill accidents, the lessons learned about the causes of accidents include:

- Mechanical accidents are numerous. Following good procedures is necessary.
- Many unwanted chemical reactions are possible. Toxic vapor clouds and toxic smoke plumes are valid
 concerns. The impacts of such accidents can potentially extend outside the mill; however, no accident
 of this nature has occurred to date.
- Product spillage within the pulp and paper mill reached adjacent waterways, causing contamination and pollution that could affect an NPP's water intake.

The consequences of pulp and paper mill accidents are often limited to a single employee being hurt by one of the mechanical or chemical processes in the mill. Farther-reaching consequences, still confined to the mill, stem from hot work conducted around flammable and explosive chemical tanks, a common occurrence across all three primary industrial processes examined for this report. In all such accidents, toxicity from inadvertent chemical reactions is a concern locally to the mill. Some accidents occurred as a result of products that spilled into adjacent rivers. One rupture of a black liquor tank caused contamination that extended outside the mill. Black liquor is a highly alkaline manufacturing residue of organic matter, NaOH (caustic soda), and other chemical products that serve as boiler fuel in the paper pulp production process.

Based on accidentology studies of pulp and paper mill accidents, the lessons learned about consequences include:

- Most accidents affected only the pulp and paper mill, its workers and contractors, and the emergency responders.
- Pulp and paper mill accidents can affect the NPP's water intake.
 - The black liquor tank rupture accident demonstrated the potential for environmental and health effects extending beyond the pulp and paper mill, including raising the pH of the river to the point that a co-located NPP using this river for intake would be forced to shut down to protect its equipment.
 - Another accident spilled "broken pulp" into the river for 13 km. These solids may cause intake screen blockage at the NPP.

5.2 Design Options and Assumptions

The HES and HTEF design options and assumptions considered for the representative NPP, HES, and HTEF are listed in Table 5-1. Assumptions are made based on physical properties and a generic geographic region.

Table 5-1. Industrial facilities design options and assumptions.

Component/Parameter	Options	Assumptions
Electrical power linkage from NPP to industrial facility	Direct linkage, load following or connection to the grid then to the industrial facility	The NPP is connected directly to the industrial facility in a behind the meter fashion.
Loss-of-offsite-power (LOOP) frequency		Default LOOP frequency is the same for the generic boiling-water reactor (BWR) and PWR model, assuming the same geographical region.

Component/Parameter	Options	Assumptions
Multiple detonations at industrial facility		Bounding accident is assumed for the first detonation overpressure.
		Ensuing detonations will not exceed bounding accident but may cause the bounding accident.
		NPP SSCs will not be adversely affected by prior non-bounding detonation overpressure events.
Blast shielding or other engineered barriers at the industrial customer other than the combined production header		Default analysis is performed without shielding.

5.3 Failure Modes and Effects Analysis

Proposed facility sizes and operations were reviewed and evaluated in a system-level FMEA for each industrial facility covered in this report. The system-level analysis does not attempt to assess the operations down to a component level. The objective is to identify hazards and their consequences. The FMEAs were performed for four perspectives. The main focus is on #1 below: NPP safety of the general public. This is the only FMEA that is used to identify the hazards that cause initiating events at the NPP. The other three FMEAs are provided for general use in determining non-radiological safety or economic impacts.

- 1. Nuclear power plant safety of the general public
 - a. This is the most important aspect of this report for licensing considerations
- 2. Industrial facility safety
 - a. Important for the industrial facility operators and nearby public
- 3. Public perception
 - a. This is important for continued operation of the two facilities and can extend beyond actual safety concerns
- 4. Economic impacts
 - a. Loss of the industrial customer or loss of the NPP energy supply adversely affect the nearby facilities.

The safety of the NPP is the focus of this report and feeds into the safety analysis decisions both for deterministic analyses to decide what hazards need to be quantified, and for probabilistic analyses to decide where the hazards fit within the PRA. The other three perspectives are provided in tabular form for the reader's information.

Each potential failure event/mode evaluated among all four perspectives were ranked with respect to severity of the event to the perspective focus, frequency of the event, and detection of the event. Each of these three categories were ranked on a scale of 1–10 then were multiplied linearly to determine a risk priority number (RPN) in which lower values indicated less risk and higher values indicated greater risk. There is no RPN cut-off at which the hazard will not be modeled in the PRA. One of the uses of the RPN scores was to identify which hazards were of most importance to eliminate through safe facility separation distance considerations. See Equation (2).

$$RPN = S \times F \times D \tag{2}$$

where S is the score for severity, F is the score for frequency, and D is the score for detection, all of which are integer values.

As much as possible, the scaling of each category was defined to minimize variability in scoring. This is detailed in Table C-1.

The flammable and detonable products and feedstocks for methanol, syngas, refinery, and pulp and paper are listed in Appendix H. The chemical properties of flash points, auto-ignition temperature and flammability limits for chemicals found in methanol, syngas, refinery, and pulp and paper mill are summarized from [2].

The toxic products and feedstocks for methanol, syngas, refinery, and pulp and paper are listed in Appendix H. The time-weighted average (TWA), short-term exposure limit (STEL), oral and dermal toxicity levels for methanol, syngas, refinery and pulp and papers are summarized from [2]. Based on the definition from OSHA [25], TWA refers to "the employee's average airborne exposure in any 8-hour work shift of a 40-hour work week which shall not be exceeded." STEL is defined as "the average exposure to a contaminant to which a worker may be exposed during a short time period (typically 15 – 30 minutes)" [26]. Most of the oral and dermal toxicity in the tables using acute toxicity estimate (ATE) as a measure to define the toxicity level. This is used to define the categories of each of the toxic materials [27].

5.3.1 Nuclear Power Plant Hazards Analysis

A group of SMEs were gathered for an FMEA to determine the hazards presented to the NPP that are not unique to the external hazards of the reference industrial facilities. The results of this FMEA informed the reference facility FMEAs discussed starting in Section 5.3.2.

The FMEAs performed for this report were all done at a high level. The intent was not to design or improve upon the generic proposed designs. The intent was to stay at a system level and concentrate on safety first above reliability and resilience.

An outline of the topics considered for the FMEA include:

- External overpressure event effects on NPP
- Industrial customer specification recommendations and assumptions for safety
 - List of industrial customers under consideration
- Thermal and electrical load effects on NPP
 - Thermal and electrical load power profiles supplied by the NPP to the industrial customer
- Hot standby mode
- Placement of the HES reboilers
- Unique risks of BWR
- Unique risks of PWR
- Production chemical routing options and effects on risk
 - Chemical storage risks
- A list of heat-transfer fluids (HTFs) under consideration and their properties.

Possible external overpressure event effects on the NPP were summarized to include the damage to the containment, damage to external coolant storage tanks, damage to switchyard components causing

LOOP, damage to above-water spray mechanisms in spray ponds, debris in spray pond or cooling tower pond, and service water pump house damage.

Note that blast overpressure-borne missiles were not assessed in this report and must be considered on a site specific basis.

Possible thermal and electrical load effects on the NPP were summarized as a load drop feeding back negative reactivity into the NPP, possibly causing a reactor trip.

The HES reboilers were considered for placement within the turbine building or in a building separate from the turbine building that would be designed and fabricated to the same requirements of the turbine building. The benefit of placement in the turbine building (if room in the existing NPP is available) is lower costs. The benefit of having its own structure is increased safety, as the FMEA results in the appendices identify.

Unique risks were considered for BWRs and PWRs for each of the hazards identified.

Industrial process production and storage were discussed as potential hazards.

5.3.1.1 List of Nuclear Power Plant Hazards Identified

The NPP-specific FMEA results are used in all the industrial facility FMEAs. All risks identified are evaluated in the sections that follow. Those not screened by an engineering evaluation are mapped into the respective ETs, and the IE frequency for these ETs are re-quantified for the respective BWR and PWR models based on the increased frequency of occurrence caused by the addition of the HES and the industrial customer at a calculated safe distance from critical SSCs.

The hazards either affected or added to the PRA by the addition of the HES and the industrial customer are listed in Table 5-2. Also listed in the table is the ET to which the hazard would map to and the status ("Included" or "Screened" from the PRA) from the FMEA panel. Potential hazards considered in adding the HES and locating the industrial customer at a calculated safe distance include a detonation at the industrial customer causing an overpressure event at the NPP site, an unisolable steam pipe leak in the HES outside of the NPP main steam isolation valves (MSIVs), a reboiler leak in the HES either causing an unisolable steam leak or contaminating the customer industrial customer steam loop, and the prompt loss of thermal load to the HES.

Table 5-2. FMEA-derived potential failures from hazards and PRA ET assignment.

Hazards	Potential NPP Process Functions Affected	Potential PRA ET Assignment	FMEA Hazard Status
Detonation at industrial customer	Loss of Offsite Power	Switchyard-centered LOOP (LOOPSW)	Screened through safe separation distance
	Loss of service water (spray pond damage or debris, cooling tower pond debris, service water pump house, forced air cooling)	Loss of Service Water System (LOSWS) (BWR) No generic PWR tree affected	Screened unless spray pond is also the ultimate heat sink

Hazards	Potential NPP Process Functions Affected	Potential PRA ET Assignment	FMEA Hazard Status
	Critical structure damage (Reactor containment, condensate storage tanks (CST), or other coolant supply tanks)	XXX-DETONATION ²	Included, but screened by safe separation distance
HES steam pipe rupture outside of NPP MSIVs	Missile damage in turbine building (if HES located in turbine building)	Main (large) steam line break in HES (MSLB- HES) TRANSIENT (MSLB- HES bounding)	Included (screened if HES is not in the turbine building)
	Main (large) steam line rupture, unisolable steam leak	MSLB-HES	Included
HES reboiler leak (Primary to Secondary Side	Large leak/rupture: Main steam line unisolable steam leak	MSLB-HES	Included
	Small leak: Contamination of the HTEF heating loop (steam or HTF)	Not a design basis event. Economic risk. BWR is a higher risk to contaminate the HTEF heating loop.	Screened for Level-1 PRA. There is an economic and environmental concern
Prompt steam diversion loss, feedback	Maximum of 13.29% thermal diversion for petroleum refinery	None. NPP can handle 30% prompt load loss.	Screened
HES steam rupture in the turbine building	Turbine building SSC damage, possible safety bus damage, depending on plant configuration	TRANSIENT, emergency power capability	Screened out by recommendation to not place HES in turbine building
General Plant Transient Due to Overcurrent from Electrical Transmission	Turbine disruption	TRANSIENT	Included
Use of HTFs instead of steam	Leak potential in heat exchanger or reboiler Fire potential in reboiler room	TRANSIENT	Not included in modeled designs for this report Properties of HTFs are listed in Section 5.4.7

5.3.2 Methanol Plant Hazards for use in FMEA

The hazards that can affect the operation of methanol plants are summarized Table 5-3, which highlights the hazards associated with syngas, methanol synthesis, distillation and purification processes.

_

² Potential new ET if a probabilistic argument is made where an evaluated overpressure damages critical structures.

The inherent properties of feedstocks, intermediate streams, and finished fuel products pose severe fire, explosion, chemical exposure, and toxicity hazards. The operating condition of the methanol synthesis process involves high-temperature and pressure hazards.

Table 5-3. Methanol hazard summary.

Process	Hazards
Syngas Production	Fire, explosion, high temperature
Methanol Synthesis	Fire, explosion, high temperature, pressure, chemical exposure, toxicity
Distillation/Purification	Fire, chemical exposure, toxicity

From Table 5-3, the hazards include fire, explosions, high temperature, pressure, chemical exposure, and toxicity. The fire and explosions hazards are relatively easy to detect based on the flame and smoke, along with the sounds from the ignition sources as described in the previous section. The temperature and pressure should be monitored in each process, and the monitoring system can report some potentially abnormal events before it becomes an accident. Chemical exposure poses some toxic concern if the undetected toxic chemicals are released to the environment. The toxic chemicals in methanol production include methanol and syngas, which are challenging to detect since they are colorless, and the odor does not have a specific distinction from other products. Leakage of methanol and syngas would lead to fire and explosion due to their high flammability.

Another concern arises when one of the feedstocks (syngas) is released to the environment. The size of the impacted area depends on the composition of the syngas. INL has ongoing research to model the syngas release and potential toxicity level using Areal Locations of Hazardous Atmospheres (ALOHA).

These hazards are analyzed in an FMEA for a nuclear-integrated methanol plant with respect to four perspectives: the NPP, the methanol facility itself, public safety and perception, and economic impact to the methanol facility. Results of the FMEA can be seen in Appendix E.

The primary mechanism of failure evaluated was methanol detonation at the facility as most recorded accidents are detonations. Methanol fires are difficult to detect visually due to its nearly colorless flame. Also, methanol vapors are slightly denser than air [28] and highly flammable [29] so it should not travel downwind significantly before ignition if released.

If methanol is combusting and generating a fire within an enclosed space, it can mature into a detonation due to build up of pressure. Other possible initiators are runaway reactions or methanation, which can lead to a sudden increase in temperature and pressure.

Although there are many hazards or effects of methanol detonation and other identified mechanisms of failure, many of them can be mitigated by siting the methanol plant at a safe distance from the plant.

Other considerations beyond the methanol reactor itself include the equipment required to process the feedstock in preparation for the reactor such as the RWGS system and the CO2 capture system using selexol solvent. For these chemical processes, the severity of the hazard can vary depending on the process conditions and mechanism of failure. If pressures, concentrations of chemicals, or temperature changes vary, the severity of the pressure build-up to detonation can vary.

5.3.3 Syngas Production Hazards for use in FMEA

The hazards of syngas production will focus on a low carbon emissions methodology of creating syngas. The feedstocks for this production will be carbon dioxide captured by selexol solvent from atmosphere and hydrogen produced from high-temperature steam electrolysis. Hydrogen production is not included as part of this accidentology analysis since it is considered that the HTEF is separate from the

rest of the syngas production facility. The syngas production facility is considered to include the selexol carbon dioxide capture and the RWGS reaction. The complete FMEA results for syngas synthesis is listed in Appendix D. The main hazards include fire, explosion, and toxicity.

The FMEA is structured to provide a comparison with hydrogen production FMEA, because syngas synthesis relies on hydrogen generated from an assumed HTEF. The similarities between syngas and hydrogen originate from the flammable nature of both gases, although hydrogen has a wider range of flammability and higher heat of combustion. A key difference between the two is that, unlike HTEF, syngas synthesis does not require steam from the NPP Therefore, a syngas production facility can be situated farther away from the NPP instead of being co-located.

A leak or fire at the syngas facility is less likely to affect the nuclear power plant. However, it is important to note that syngas is a denser gas than hydrogen. While leaked hydrogen is dispersed easily into the atmosphere, syngas may be carried by the wind while undergoing a slow diffusion process. There is some probability that the wind could blow in the direction of the NPP, transporting leaked syngas. If the syngas concentration is above the lower flammability limit and it meets an ignition source at the NPP complex, a syngas fire can occur there. Additionally, syngas also poses a toxicity hazard due to its carbon monoxide content, which can restrict outdoor operations even if there is no fire.

5.3.4 Refinery Plant Hazards for use in FMEA

The FMEA for a nuclear-integrated refinery has been performed for the four perspectives, as mentioned above and documented in Appendix E.

For NPP safety, the hazards from the co-located refinery plant to the nearby NPP are analyzed and included in the FMEA in addition to the hazards associated with the NPP operation itself. For the refinery hazards, both actual historical accidents and hypothetical events discussed with SMEs are included in the FMEA. For public perception, all the events in the FMEA from NPP safety and refinery are analyzed. The frequency and detection are the same as those assigned in NPP safety and refinery, but the severity is different based on the level of the public concerns that arise for each of the event. A similar approach is used for the FMEA of economic impacts, where different severities ranging from 1 to 10 are assigned based on the potential revenue losses associated with each event.

Petroleum refineries are complex, high-valued facilities that process large volumes of flammable crude oil to produce large volumes of product fuels. To operate profitably and safely under environmental policies and constraints, refineries efficiently integrate steam and power demands within all the refining processes into a single, self-sufficient process. Each step along the refining pathway, from raw material storage to finished fuel production and storage, includes multiple processes that may pose a threat to the facility, environment, and workers as well as the residents close to the facilities. The inherent properties of feedstocks, intermediate streams, and finished fuel products pose severe fire, explosion, chemical exposure, and toxicity hazards. The operating condition of the refinery process involves high-temperature and pressure hazards. Table 5-4 summarizes the hazards associated with the refining processes, including crude processing, intermediate stream conversion and upgrading, component blending, and product storage.

Table 5-4. Refinery hazard summary.

Process/Storage Unit	Hazards
Crude Oil Fractionation	Fire, high temperature, chemical exposure
Coking	Fire, high temperature, toxicity
Fluid Catalytic Cracking	Fire, high temperature, toxicity

Process/Storage Unit	Hazards
Hydrotreating/Hydroprocessing	Fire, explosion, high temperature, pressure, chemical exposure, toxicity
Alkylation	Fire, explosion, high temperature, pressure, chemical exposure, toxicity, corrosive chemicals
Sulfur Plant	Fire, explosion, high temperature, chemical exposure, toxicity
Hydrogen Plant	Fire, explosion, high temperature
Fuel Gas Treating	Fire, explosion, chemical exposure, toxicity

From Table 5-4, the hazards for the refinery process include fire, explosion, high-temperature exposure, chemical exposure (e.g., hydrogen sulfide, naphtha), and overpressure events. Sensors can detect when chemicals leaks occur and can, in turn, provide the staff with time to ignite flares and prevent a larger accident, shut down equipment safely, and evacuate the area. However, there are many cases of documented events, especially fires and detonations, that have occurred without sufficient warning. Fire, explosions, high-temperature exposure, and overpressure events may impact the facilities and staff in both refinery plants and co-located NPPs depending on the distance from and location of the ignition points and the availability of the safety systems. The consequences of these events can include the loss of lives, injuries, and damage to the industrial facility. Fire or explosions from the refinery site occur due to hot work around tanks and ignition sources present when a leak occurs. Sometimes the leak cannot be detected fast enough to prevent an accident. Once the accident occurs, it can be easily detected on-site because the ignited fire and explosions usually come with flame and smoke or sounds. Chemical exposure poses a concern when undetected toxic chemicals are released to the environment. It is best to rely on electronic sensors to alert on-site staff of toxic chemical presence. Carbon monoxide, syngas, and hydrogen sulfide are examples of toxic chemicals found in the industrial sites under consideration. Hydrogen sulfide is an example of a toxic chemical that can be detected easily on-site by sensors due to its specific odor, even though it is colorless [2]. However, it is toxic at low levels of concentration in air and failing to detect the hydrogen sulfide poses a health hazard for workers, first responders, and residents nearby. While the hydrogen leakage does not cause long-term impacts on the health of the on-site staff and nearby residents, its high flammability can cause potential fires and explosions. If these events occur in close enough proximity, and with enough severity to affect the NPP, there may also be loss of lives, core damage, radioactive chemical release, economic loss, and impacts on public opinion of NPP safety.

5.3.5 Pulp and Paper Mill Hazards for use in FMEA

The hazards for pulp and paper plants in terms of storing the feedstocks, processing, and finished product storage areas are summarized in Table 5-5, which highlights the potential hazards associated with during the process or locations in the facilities.

Table 5-5. Full-process kraft mill pulp hazards summary.

Process or Location	Hazard
Woodyard	Fire
Storage	Fire
Bale and Finishing	Fire
Digesting	Explosion, chemical exposure
Recovery Boiler	Explosion, fire, chemical exposure
Paper-Making Machines	Fire
Power Plant	Fire
Bleaching	Explosion, chemical exposure

Finishing	Fire, chemical exposure
Converting	Fire

From Table 5-5, the hazards include fire, explosion, and chemical exposure. The fire and explosions hazards are relatively easy to detect based on the flame and smoke, along with the sounds from the ignition sources. Chemical exposure poses both toxic and non-toxic concerns.

The FMEA for a pulp and paper mill neighboring an NPP is divided into four segments identifying potential failure modes within the mill operations to assess the potential effects of these failures on the adjacent NPP, the pulp and paper mill itself, public impact and perception, and the economic wellbeing of the operation. The FMEA results are found in the tables of Appendix G. Facility processes were systematically evaluated to uncover any potential for failure that could lead to downtime, compromise safety, or impact the NPP, public health, and perception, or the economics of the facilities. Multiple facets of the mill's operations were considered, from the mechanical aspects of the pulp processing machinery to the chemical treatment stages and waste management systems.

Pulp and paper mills are prone to several hazardous incidents, with explosions, fires, and chemical exposures or leaks among the most severe. Recovery boilers, where chemicals are burned to recover pulping chemicals, are especially explosion-prone areas due to the high-pressure conditions and volatile substances involved. Fires are potential risks in multiple areas of a pulp and paper facility, such as the woodyard or inside the paper machines where overheating equipment can ignite paper products. Chemicals like the bleaching agent chlorine dioxide and the pulping byproduct "black liquor" pose particularly concerning health risks to workers and the environment when mishandled. Accidental releases of these substances can lead to toxic exposure, causing severe injury or fatality and can have devastating environmental impacts if they enter waterways.

The FMEA conducted on a pulp and paper mill near an NPP has identified various hazards. Some hazards, such as machinery malfunctions and localized chemical exposures, are mainly contained within the mill. However, others have the potential to extend beyond the mill's boundaries. For example, the accidental explosive potential of recovery boilers and digesters could exert enough force to impact nearby structures. Additionally, the use of hazardous chemicals such as chlorine dioxide or black liquor not only poses acute health risks to employees in the event of a leak but also brings the risk of environmental contamination. If these chemicals were to contaminate shared water sources, the effects could extend to public health and the NPP. Although these issues are concerning, establishing a safe siting distance could help prevent potential consequences from reaching the NPP.

5.4 Nuclear Power Plant Safety-Critical Structures

The reactor building is the primary critical structure at an NPP. It is also the most well-protected from any external forces, such as blast impulse shock waves. Nuclear-grade concrete walls encase the containment and provide significant protection from external forces to the reactor internal structures in addition to providing significant protection from accidental release of ionizing radiation. Critical structures external to the reactor building are typically designed to withstand postulated extreme local wind and seismic loads. These include refueling water storage tanks and condensate storage tanks (CST). No attempt was made to evaluate missiles created by an industrial facility overpressure event because that is a site specific analysis.

5.4.1 Reactor Containment Structure Fragility to Overpressure Events

Reactor building concrete walls were characterized in EGG-SSRE-9747, "Improved Estimates of Separation Distances to Prevent Unacceptable Damage to NPP Structures from Hydrogen Detonation for Gaseous Hydrogen Storage" [30]. The lowest static pressure capacity of nuclear concrete identified is 1.5 psi. This conservative estimate was used for the blast analyses performed in prior hydrogen plant

separation studies by INL [31],[32], and is adopted as the static pressure capability of nuclear concrete walls in this study as well.

NRC Regulation Guide 1.91 [3] uses a 1.0 psi overpressure when calculating safe standoff distances from potential explosion sources to the nearest NPP SSC.

5.4.2 Safety-Critical External Structures Fragility to Overpressure Events

Critical structures outside of the reactor building have been identified when assessing external events such as high-wind fragility for PRA.

External water tanks are located close to the reactor building to provide condensate storage and coolant for routine, refueling, and emergency operations. In some cases, there are concrete walls placed around the external tanks for protection, but some NPPs choose not to include external protection other than the tank's own construction. These tanks are built to extreme standards. According to Reference [33] and other individual plant examinations of external events, they are equivalent in structural integrity against wind pressure to a Category I Structure. This means that the tanks are nearly as durable as the reactor building itself and nearly as durable as reactor containment when it comes to handling pressure. The CST and other storage tanks are assumed to be Category II structures when considering susceptibility to wind missiles.

Service water intakes are solid structures, and their failure modes typically involve the buildup of debris on the screens instead of physical damage; and the pump house is typically built to withstand tornadic or hurricane winds. In some NPP PRAs, a loss of service water is itself an initiator that challenges the NPP to shut down safely.

Loss of switchyard components means a loss-of-offsite power (LOOP) event that challenges the NPP to shut down safely. Switchyard components are sensitive to wind pressure, and particular care needs to be taken to ensure facility location provides a safe separation distance between the source of an explosive overpressure event and these SSCs.

5.4.3 Non-Safety-Critical External Structures

In addition to critical structures, some other structures that affect operations, but not typically the ability to safely shut down the reactor, are located in the plant yard as well: circulating water and standby service water pump houses, demineralized water storage tank(s), cooling towers, well water pump houses, liquid nitrogen tanks, and hydrogen and nitrogen gas cylinders, which present stored energy in the form of chilled and pressurized gas.

Further, the day-to-day NPP operations would be affected by damage to the turbine building, administrative building, and maintenance support buildings located throughout the site.

5.4.4 Heat Extraction System Unisolable Steam Pipe Rupture

A large steam line break is the most common hazard introduced by adding the HES to the NPP. There is one non-nuclear safety (NNS) related isolation valve immediately after the steam tap for each of the HES designs listed in Section 4.1. Although not credited in any accident analysis response scenario based on its NNS classification, the success of this valve is the first line of defense of a steam line rupture within the HES after the NPP's MSIVs have failed to isolate. Isolation and control valve ruptures are also a possibility that need to be modeled. After the isolation valves, all the other active components listed in Section 4.1 are evaluated in the HES FTs (Sections 6.2 and 6.3). The FT result was added to the IE for a large steam line break, as described in Section 6.2.1 for a PWR and Section 6.3.1 for a BWR.

Seismic considerations were also added to the IE for a large steam line break. This includes loss of function of the valves due to a seismic event. The PRA logic includes options for seismic events in five bins ranging from a peak ground acceleration of 0.17 to 2.12 g. Bin frequencies and gamma uncertainty

distribution parameters utilized are from the NRC generic BWR and PWR models. These are reported in Table 5-6.

Table 5-6. Seismic bin	peak ground	accelerations	and frequencies.

Bin #	Peak ground acceleration (g)	Frequency (/yr)	r of gamma
1	0.17	7.23E-05	3.00E-01
2	0.39	6.49E-06	3.00E-01
3	0.71	2.29E-06	3.00E-01
4	1.22	2.74E-07	3.00E-01
5	2.12	9.60E-08	3.00E-01

Extensive searches on seismic fragility constants were performed and the best data found was for residual heat removal motor operated valves and feedwater check valves from Reference [34]. The fragility constants and the valves they were applied to are documented in Table 5-7.

Table 5-7. Seismic fragility constants for valves evaluated in main steam line break.

Makes Toma	Seismic Lo	Seismic Lognormal Fragility Constants		
Valve Type	Am (g)	βr	βu	
Gate valve as a motor operated valve (MOV)	3.10	0.24	0.37	
Check valve (CKV)	1.40	0.34	0.30	
Flow control valve (FCV)	3.10	0.24	0.37	

5.4.5 Heat Extraction System Reboiler Leak

Two types of reboiler leaks are considered for the PRA: a slow leak that is not a prompt safety concern to the NPP operation and a reboiler rupture. The reboiler faults are considered equivalent to heat exchanger faults for the purpose of this PRA. The construction of a reboiler is more of a teakettle design than a tube-and-cartridge heat exchanger design. A reboiler design is more durable than a tube and shell heat exchanger, so using the extensive heat exchanger failure data is considered conservative in place of the lack of operational data found for reboilers.

Slow Leak of an HES Reboiler: The heat-transfer loop from the reboiler(s) to the industrial facility will always be operating at a lower pressure than the NPP steam loop through the HES. This prevents the contamination of the NPP steam loop if the deionized water loop coming back from the industrial facility were to somehow be fouled. Small leaks in the reboiler may contaminate the heat-transfer loop to the industrial facility. This can cause a cleanup problem if there is enough activity transferred to the heat-transfer loop. This is a unique potential hazard to the LWR NPPs considering this modification. There are prevention, detection, and mitigation measures that obviously would need to be in place to monitor for and react to any small leaks including routine isotopic chemistry sampling. This hazard could potentially cause steam loop isolation of the industrial facility and resultant economic issues during reboiler repair and unlikely, but possible cleanup of the industrial facility steam supply.

It should be noted that, although not detailed in this study, ongoing studies related to use of this basic reboiler barrier design to generate steam to offsite industrial users assume a second series reboiler with a high pressure intermediate pressurized water loop to provide multiple barriers for offsite release due to

heat transfer equipment cross-boundary leakage. This study is concerned with reactor safety and did not consider the architecture of a fully representative heat transfer system.

Rupture of an HES Reboiler: Depending on the size of the supported facility, there can be up to three HES reboilers. An HES heat exchanger rupture failure maps to the HES large steam line break event and is treated as an event within the IE FT for PWRs (Section 6.2.1) and BWRs (Section 6.3.1).

5.4.6 Prompt Steam Diversion Loss Causes Feedback

The addition of the HES to the NPP provides a new steam loop that must be evaluated for safety. The designs considered for this study assume that the amount of steam diversion is limited to 13.3% of the total steam production (Table 4-1). This screens out one of the postulated hazards that the prompt load drop was felt by the NPP and pushed to the turbines, even with the successful closing of the HES isolation valves. The FMEA team determined that LWR NPPs can withstand up to a 30% load drop without having to trip.

5.4.7 Use of Heat Transfer Fluids and Ignition Potential

The use of steam as the heat-transfer medium screens this hazard out from consideration. HTFs have desirable qualities of consistent thermal storage for longer distances and periods of time than steam. While steam is the medium of choice of most NPP operators interviewed, there is a possibility that HTFs will be considered. Four representative HTFs with a range of operating temperatures and states are listed in Table 5-8: Therminol 66, Dowtherm A, Dowtherm G, and Therminol VP-1. HTF ignition would result from a leak with an ignition source at a temperature above the flash point or overheating the HTF to the auto-ignition temperature in the presence of oxygen. HTF leakage probability was not determined for this study.

A leak and fire within the reboiler building could damage the equipment and cause the NPP to isolate the HES. If the fire is severe enough, there is a possibility of damaging the ability to isolate the HES without closing the NPP's MSIVs.

Heat-Transfer Fluid	Max Operating Temperature (°F)	Flash Point (°F)	Auto-ignition (°F)	
Dowtherm A	494 (liquid) 495–750 (vapor)	236	1110	
Dowtherm G	675 (liquid)	280	810	
Therminol 66	650 (liquid)	338	705	
Therminol VP-1	256 (liquid) 257–750 (vapor)	230	1114	

Table 5-8. Heat-transfer fluid properties.

5.5 Industrial Facility Siting Analysis

The placement of the industrial customer is determined first and foremost by the safety of the NPP and the public. Other considerations are made due to the geographical properties of the existing NPP site, the proximity to the reboiler building to make the steam supply line as efficient as possible, and the accessibility of the industrial customer for transport of the final product. The following sections provide analyses useful to visualizing the inherent risk evaluation aspects for industrial customers considered in this report, the standoff distances required for these hazards and plant sizes, and where in the industrial customer facilities these hazards are located.

It is noted that operating U.S. NPP's were all originally evaluated for the risks of nearby industrial facilities such as in RG 1.91 [3] and other then-accepted NRC methodologies under original licensing agreements. Similar risk analyses as originally agreed upon will be needed for assumed nearby location of new industrial facilities that may be sited to maximize energy transfer from a NPP in the form of electrical, heat, or hydrogen to new industrial users. The sections that follow identify topical evaluation areas but are not necessarily intended as approved approaches that will be accepted by the NRC.

5.5.1 Blast Analysis

The major accidents in industrial installations are usually related to a loss of containment that releases hazardous materials. Following the discharge, how the situation unfolds will be influenced by the physical form of the released substance and other factors, such as the volume of the substance spilled and prevailing weather conditions. Figure 5-6 shows the possible scenarios following a hazardous material release [35].

A liquid spill can contaminate the soil and/or body of water. It can also evaporate or catch fire if it is flammable and it meets an ignition source, probably by igniting the vapor cloud. The combustion can release smoke, thermal radiation, and overpressure. Alternatively, a flammable or toxic cloud may develop if no immediate ignition occurs. The flammable cloud can ignite and produce a flash fire and thermal radiation hazard. Depending on the amount of material and degree of confinement, a flash fire may lead to an explosion, causing overpressure and missiles.

Meteorological conditions, including wind, can contribute to the creation of a toxic vapor cloud. When a hot, pressurized liquid is emitted into the atmosphere and instantaneously vaporizes, it often results in a vapor-liquid blend that can lead to a dense vapor cloud due to the evaporation of liquid droplets, thereby elevating the concentration of the vapor in the air.

A gas or vapor release can lead to cloud formation if the release velocity is low. However, if the release is at a high velocity, the resulting air entrainment will dilute the mixture, causing it to disperse in the atmosphere making the formation of a flammable cloud unlikely. Should ignition take place, there is a risk of a jet fire occurring in both scenarios.

Dust released into the atmosphere can pose hazards such as allergenic reactions. Additionally, fine dust can lead to severe explosions if dispersed in the air within an enclosed space. These explosions typically happen inside equipment like silos, dryers, or cyclones, rather than from a containment breach, but their impacts can still extend over a large area.

If the pressure within a pressurized tank exceeds a certain threshold or if the tank's integrity is compromised due to high temperatures from a fire, an explosion can occur. This explosion would impact the surrounding area and could launch debris over great distances. Should the contents be flammable, it's likely that the explosion, which might be a Boiling Liquid Expanding Vapor Explosion (BLEVE), would be accompanied by a fireball.

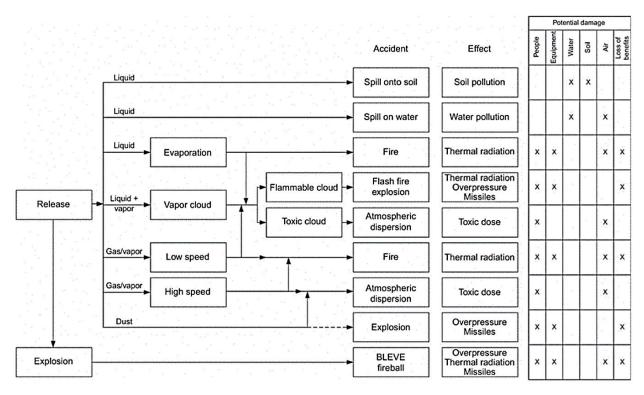


Figure 5-6. Schematic representation of possible accidents following a loss of containment [35].

Industrial facilities are designed such that fire and explosion hazards are limited within their safe boundaries. However, vapor clouds have the potential to traverse outside the boundaries before meeting an ignition source. Therefore, special consideration is given to atmospheric dispersion and vapor cloud explosions (VCEs) in this Section.

For a VCE to happen, certain criteria must be met [35]. The substance should be combustible, and the ignition must be delayed for a cloud of fuel-air mixture to form. Otherwise, an immediate ignition causes a jet fire instead. A portion of the cloud mixture must also fall within flammable limits, making it capable of ignition. The vapor cloud additionally needs to reach a minimum size, and there must be turbulence present, which can be caused by the manner of the release, such as a jet, or by interaction with obstacles that lead to partial confinement. This confinement and congestion are important because they are commonly present in industrial installations, which creates favorable conditions for slow deflagrations to accelerate in what is known as flame acceleration resulting in more severe explosive cases [36].

The mechanical energy from an explosion generates an overpressure wave that travels through the atmosphere at a specific speed. This wave is created by the opposing forces of the increasing pressure from combustion and the decreasing pressure caused by the expanding gases. There are various methods to calculate this blast overpressure, with the empirical models being the most popular options due to their simplicity and reliable results [36]. Empirical models, which are based on data from numerous experiments, allow for quick calculations of pressure and impulse from explosions using layout and thermodynamic information about the flammable mixture. They are particularly useful in the preliminary design phase of new facilities, which is the scope of this current study. However, many of these models do not provide direct guidance to assess deflagration to detonation transition (DDT) likelihood, which may lead to underestimations of explosion severity. Therefore, it is recommended that additional methods are used to evaluate DDT events on mature, site-specific industrial facility designs.

Among the various empirical methods to calculate unconfined blast overpressures, three methods are of particular interest in this study: the Bauwens-Dorofeev method, the TNT equivalent method, and the Baker-Strehlow-Tang (BST) method. These methods use separate families of empirical blast curves.

The Bauwens-Dorofeev method calculates the blast overpressure based on the amount of detonable mass within the cloud. A key feature of interest in this method is that it has empirical polynomial equations to calculate the detonation cell size and eventually the detonable mass of common flammable gases, including hydrogen, methane, and propane [37]. The available hydrogen data is extensive, which gives a good confidence in hydrogen blast overpressure calculations. The detonable mass (m_{det}) is used to calculate the detonation energy (E_{det}) according to Equation (3), where H_c is the heat of combustion. Detonation energy is used to calculate a set of dimensionless distance ($R^*_{Bauwens}$) from the center of the detonable region according to Equation (4), where R is a set of distance values and $P_{ambient}$ is the ambient pressure. This set of dimensionless distance is used to calculate the corresponding set of scaled overpressure (P^*) using the empirical Equation (5) [37].

$$E_{det} = m_{det} H_c \tag{3}$$

$$R_{Bauwens}^* = R \left(\frac{P_{ambient}}{E_{det}}\right)^{1/3} \tag{4}$$

$$P^* = \frac{0.34}{(R^*)^{4/3}} + \frac{0.062}{(R^*)^2} + \frac{0.0033}{(R^*)^3}$$
 (5)

The TNT mass equivalence method is the simplest means of modeling VCEs. It works by finding the equivalent mass of TNT containing the same energy as the combusted fuel [37]. The interest in this method is that it is prescribed by existing nuclear regulation to calculate the safe distance at which the overpressure drops to 1 psi [3]. This 1 psi limit is also prescribed in the Environmental Protection Agency (EPA) citing the U.S. Code of Federal Regulations for chemical accident prevention [38], although its quantification methodology is not specified. The TNT equivalent mass is scaled by an equivalence factor (Fequiv) as shown in Equation (6), often also called yield factor, efficiency, or efficiency factor. In a sense, this TNT equivalency informs the efficiency of energy conversion from chemical combustion into mechanical blast. The theoretical maximum equivalency is 40%, however the empirical equivalency is proposed between 1 to 20% according to different authors [39]. The NRC adopts equivalency factors from FM Global [40] (i.e., 5% for unconfined combustible gases and vapors such as hydrogen, 10% for unconfined combustible dusts, and 15% for unconfined ignitable fibers [3]). Meanwhile, the EPA prescribes an equivalency of 10% for flammable gases and liquids [38].

$$E_{TNT} = F_{equiv} E_{fuel}$$

$$m_{TNT} H_{c,TNT} = F_{equiv} (m_{det} H_c)_{fuel}$$

$$m_{TNT} = F_{equiv} \frac{(m_{det} H_c)_{fuel}}{H_{c,TNT}}$$
(6)

The TNT mass equivalent is used to calculate a set of scaled distances (R^*) using Equation (7), and the scaled overpressure (P^*) is found from an empirical curve relating P^* to R^* .

$$R_{TNT}^* = \frac{R}{m_{TNT}^{1/3}} \tag{7}$$

The BST method assumes that only the parts of the flammable cloud that are congested or partially confined contribute to the overpressure buildup [35]. The appeal of the BST method is that it is one of the few empirical methods that provides direct guidance to assess DDT likelihood [36], which is when the flame speed reaches at least Mach 5.2 [41]. BST analysis consists of these steps [35]:

- 1. Calculate the volume of a cloud containing the mass of fuel at the stoichiometric concentration.
- 2. Identify the volume of the congested or partially confined portion of the flammable vapor cloud.
- 3. Calculate the explosion energy (E) by multiplying the volume of the congested or partially confined portion of the flammable vapor cloud by 3.5 MJ/m³.
- 4. Calculate the set of scaled distances (R*) from the center of the explosion using Equation (8).

$$R_{BST}^* = \frac{R}{(E/P_0)^{1/3}} \tag{8}$$

5. Select the appropriate flame speed (Mach number) from the values listed in Table 5-9 based on the fuel and congestion levels.

Table 5-9. Flame speed Mach numbers (M_f) of BST method.

Fuel reactivity		Congestion level		
	Low	Medium	High	
High: hydrogen, acetylene, ethylene oxide, propylene oxide	0.36	DDT	DDT	
Low: methane, carbon monoxide	0.026	0.23	0.34	
Medium: all other gases and vapors	0.11	0.44	0.5	

- 6. Use the BST empirical curves to obtain the dimensionless peak side-on overpressure based on R^*_{BST} and M_f from steps 4 and 5.
- 7. Convert the dimensionless side-on peak overpressure to the peak side-on overpressure by multiplying it by the atmospheric pressure.

A comparison of blast overpressure and safe separation distances between the Bauwens-Dorofeev and the TNT equivalence method was conducted in a previous study for the pre-conceptual hydrogen HTEF designs under consideration [1]. The results summarized in Table 5-10 show that the TNT equivalence method prescribed by RG 1.91 [3] are more conservative than the hydrogen jet-leak specific Bauwens-Dorofeev method.

Table 5-10. Distance to 1.0 psi for maximum hydrogen detonation scenarios for HTEF sizes as calculated using Bauwens-Dorofeev and TNT equivalent methods.

WEED CO. (MAN)	Safe Distance (m)		
HTEF Size (MW _{nom})	Bauwens	TNT	
100	61	81	
500	168	204	
1000	215	252	

Hydrogen is lightest element and is therefore dispersed easily into the atmosphere upon release. In contrast, other flammable gases may be closer to the density or denser than air and therefore have the potential to be transported downwind before they combust. For that reason, a combined analysis of

atmospheric dispersion and combustion is needed to estimate these other detonation overpressure hazards to the NPP. Section 5.5.3 presents an example case of this combined analysis. Note that the atmospheric dispersion analysis is highly sensitive to site characteristics such as topology and meteorological conditions. As the current study does not target a specific site, a thorough analysis of all related industrial facilities is not performed, and only an example case study is presented. This is different for hydrogen detonation which does not require atmospheric dispersion analysis and was therefore performed in more detail.

An energetic detonation of a downwind dispersed vapor cloud is highly unlikely. However, the presence of environmental confinement and congestion may lead to flame acceleration that increases its overpressure, and in certain extreme conditions lead to a DDT. Therefore, the combined analysis in this report utilizes the BST methodology to account for these possible scenarios.

5.5.2 Blast Mitigation Strategies

Blasts that are attenuated or suppressed can be considered in many of the codes and standards that are used in fire protection plans and other regulatory codes and regulation guidelines. It is common practice in industry to place engineered barriers where appropriate.

The detonation overpressure analysis we have performed so far is for unattenuated blasts, based on the conservative assumption that there is a direct line of sight between the detonation source and the target. In practice, blast energy may be attenuated either through natural barriers such as hills, vegetation, and engineered mitigation techniques. This section summarizes several engineered methods to mitigate blast overpressure from a review study performed by SNL for this report [42]. The reference discusses three categories of techniques to mitigate overpressure energy: isolation, suppression, and attenuation.

Explosion isolation techniques aim to safeguard equipment not directly hit by an initial explosion from subsequent blasts. Mechanically, active valves shut upon explosion detection, while passive valves respond to overpressure to block a flame front from propagating to other pipe sections, as illustrated in Figure 5-7. Chemical suppression, such as the release of a chemical suppressant into pipes, is another method to halt explosion flame fronts. These methods are proactive measures for internal protection, but may be less relevant in hydrogen facilities where explosions are more likely to occur outside vessels due to the absence of oxygen inside the system. Even so, they could still offer some protection if an external blast causes flame propagation within the piping network.

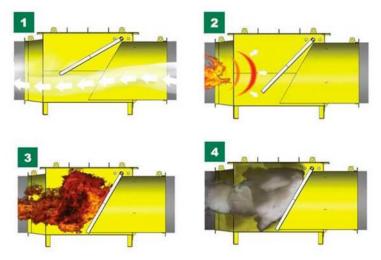


Figure 5-7. Example of a passive explosion isolation valve [43].

Blast suppression techniques are designed to either prevent ignition or slow down the flame front after ignition, with broader applications than isolation methods, extending beyond piping networks. Water mists are one such suppressant, with varying opinions on their effectiveness. Two main mechanisms are proposed: momentum transfer from larger water droplets and blast energy dissipation via the evaporation of smaller droplets, also known as quenching, which can dilute the fuel-air mix to safe levels. While some researchers see both mechanisms as valid for blast mitigation, others emphasize one over the other, citing differences in shockwave properties.

Beyond water, studies have investigated two-phase chemicals and powders as explosion suppressants. Aqueous foams can lower peak overpressure from explosions, as demonstrated in an experiment where a detonation in a plastic tent was suppressed by an aqueous foam. Additionally, commercial solutions like dry powders are available, which can be deployed into a vessel upon explosion detection to control the blast, as illustrated in Figure 5-8.



Figure 5-8. Example of an explosion suppression technique [44].

While aqueous chemicals and dry powders can mitigate blasts, applying them in a hydrogen plant is complex. Effective mitigation usually requires the explosive area to be entirely engulfed in the suppressant, which is not feasible for unpredictable hydrogen leaks. Spraying these substances when an explosion looms may not replicate tested methods and could worsen the explosion by causing more turbulence and mixing hydrogen with air. Additionally, such suppressants could be costly, difficult to clean up, and potentially harmful to the environment and nearby people, especially if granular materials are propelled by the blast. Water is considered a more suitable suppressant for hydrogen plants, provided the system design supports blast mitigation through a water deluge system.

Blast attenuation techniques aim to redirect the blast wave energy away from targets and the protected population. This objective is typically achieved by using barriers like blast walls, which utilize reflection, absorption, and diffraction to attenuate the energy that passes through the barrier.

Solid blast barriers, commonly used in oil, gas, and chemical industries, offer protection from overpressure events and propelled projectiles, with limited information on their use in hydrogen plants. NFPA 2 [17] discusses the application of blast walls in hydrogen facilities for safeguarding equipment and structures. These barriers can be either freestanding or part of existing infrastructure and are typically made of concrete or steel, with modular designs allowing for reconfiguration. Thicker concrete walls and steel reinforcement can enhance barrier durability and blast resistance. The wall's height and distance from the explosion can influence its protective efficacy; taller walls may increase overpressure within the

blast area but provide more external protection, while proximity to an explosion impacts the level of overpressure and temperature experienced. Figure 5-9 shows two types of blast barriers made of concrete and metal.





Figure 5-9. Blast barriers made of modular concrete blocks (left) [45] and metal (right) [46].

Solid blast barriers can also be deployed in alternate geometries other than straight vertical walls, such as tall and thick parallelepiped, trapezoidal, triangular, and cylindrical barriers. Key geometric parameters for blast barriers include height, thickness, inclination angles of the front and rear faces, and the barrier's position relative to the blast source. Maximizing height and thickness within space and financial limits is recommended to enhance the barrier's shock wave interaction surface. The inclination angle of the wall's upstream face influences overpressure attenuation and the wall's load. A 90° inclination angle close to the blast source screens overpressure effectively but also bears a high load. Despite this, right-angle inclinations may still be preferred for their space efficiency compared to smaller angles.

Porous barriers have also been proposed, such as using metal perforated plates, chain mails, and woven wire meshes. These porous barriers can be deployed in layers to improve their efficiency.

Other materials have been investigated to act as blast barriers, including water, granular materials (e.g., sand, rock particles, polystyrenes) and sacrificial claddings. Thin plastic bags can be filled with water within a steel frame to reduce overpressure and impulse downstream. The water wall's effectiveness improved with increased height and proximity to the blast origin, mirroring the properties of a solid barrier. This solution could be cost-effective, involving only a steel frame, plastic, and water, but the water would need replacement after an event. A potential drawback is that the blast could propel water droplets that could injure people even if they are protected by the water wall. Alternatively, a water wall can be formed by a water curtain over a chain mail grid to reflect the shock wave and reduced downstream overpressure and impulse. Granular materials can also be used to attenuate shock waves. They are more effective when they consist of smaller particle diameters and are extended in length along the path of the shock wave.

Sacrificial claddings, which consist of a crushable core between two thin plates, are unique both in geometry and material compared to traditional blast barriers. Upon encountering a blast wave, the front plate moves toward the rear plate, causing the core to plastically deform and absorb kinetic energy, thereby reducing the overpressure transmitted to the rear plate and beyond. The core is typically made of a cellular material like polyurethane foam that can withstand significant plastic deformation. The cross section of this cladding is shown in Figure 5-10.

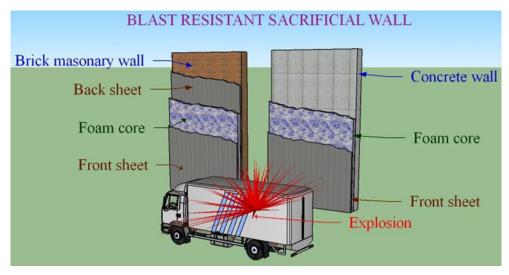


Figure 5-10. Illustration of a sacrificial blast wall [47].

Such claddings are effective in reducing overpressure and could be used for blast mitigation in hydrogen plants. However, they are single-use due to the permanent deformation from blasts and would require replacement after an explosion, potentially making them more costly than more durable alternatives. The effectiveness of sacrificial claddings also depends on having a core of adequate thickness, which varies with the unpredictable magnitude of potential blasts at a facility. Underestimating the blast load could result in insufficient protection, while overestimating it could lead to unnecessary material, cost, and space usage.

The engineered mitigation methods described above should not be the first priority in ensuring safety. Reference [42] discusses the hierarchy of control in safety management illustrated in Figure 5-11. The figure shows that the most effective way to manage hazards in hydrogen facilities is to eliminate the risk of events. This can be done by preventing hazardous gas or liquid leaks using appropriate equipment and materials, and by implementing strict leak detection, inspection, maintenance, and repair procedures. Additionally, ignition sources can be eliminated by using properly rated electrical equipment and ensuring proper grounding and bonding, as well as by enforcing no-smoking policies and providing appropriate training.

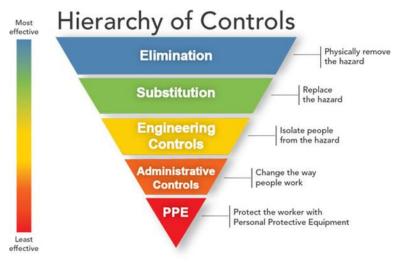


Figure 5-11. Hierarchy of controls.

After efforts to eliminate the hazard, engineering controls such as blast suppression techniques can be used to quench an explosion before it spreads. Should an overpressure event still occur, engineering controls can also isolate the hazard from people and infrastructure through blast isolation and attenuation techniques. Finally, personal protective equipment is considered the least effective means of protection in the hierarchy of controls.

5.5.3 Atmospheric Dispersion Analysis

Atmospheric dispersion analysis is needed for industrial facilities that may emit dense hazardous gases during normal operations and/or accident conditions. The purpose of this analysis is to estimate the extent of hazards extending beyond the industry perimeters that may disrupt the safe and secure operation of NPPs and/or public wellbeing. The Gaussian advection-diffusion model is used in this analysis with possible benchmark with other models in future studies. This study uses the ALOHA [41] [48] free software tool by developed by the EPA. ALOHA adopts the Gaussian dispersion model which is commonly used for accident response planning. Benchmark analysis using other dispersion models may be done in future studies.

5.5.3.1 Dispersion Modeling in ALOHA

The Gaussian model suggests that, as the distance downwind grows, the concentration profile of a continuous release of gas with neutral buoyancy will converge toward a Gaussian distribution as illustrated in Figure 5-12. The gas plume diffuses along the y and z axes to converge toward a Gaussian distribution as it is transported through advection along the x-axis direction. Longer measurement averaging periods not only encourage a Gaussian configuration but also expand the spatial extent of the distribution.

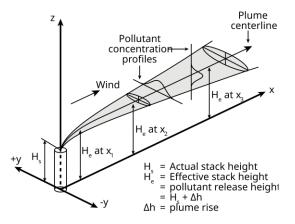


Figure 5-12. Illustration of Gaussian dispersion model [49].

The concentration of a gaseous mixture following a short release is given in Equation (9) as follows [41]:

$$C(x, y, z, t) = \begin{cases} \frac{\chi}{2} \left[erf\left(\frac{x}{\sigma_x \sqrt{2}}\right) - erf\left(\frac{(x - \bar{u}t)}{\sigma_x \sqrt{2}}\right) \right] & (t \le t_r) \\ \frac{\chi}{2} \left[erf\left(\frac{x - \bar{u}(t - t_r)}{\sigma_x \sqrt{2}}\right) - erf\left(\frac{(x - \bar{u}t)}{\sigma_x \sqrt{2}}\right) \right] & (t_r < t < \infty) \end{cases}$$
(9)

Where σ_x is the dispersion parameter and t_r is the duration of the release.

ALOHA uses the Gaussian dispersion model of continuous air pollution flumes, and the heavy gas model for gases or aerosols that are heavier than the surrounding air. This heavy gas model is illustrated in Figure 5-13. Initially, a heavy gas cloud will settle away from its origin point in every direction due to being denser than the ambient air. Subsequently, the cloud moves in the direction of the wind, resembling the flow of water, driven by the combined effects of wind force, gravitational settling, and its own momentum. As the movement of the dense gas cloud persists in the wind's direction, it mixes with the surrounding air, which dilutes and decreases its density. Once sufficiently diluted, the cloud eventually acts like a gas with neutral buoyancy.

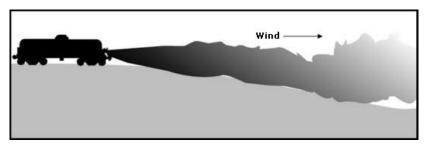


Figure 5-13. Illustration of heavy gas dispersion model [41].

The ALOHA software employs the BST approach to determine the overpressure caused by the ignition of flammable gases. This approach presupposes that the explosion originates within the zone where facilities handling combustible vapor are densely located. An integral aspect of the BST method is its capacity to modify the Mach number of the flame speed and the resulting overpressure in correlation with the obstacle density surrounding the point of ignition. For modifications of the flame speed, reference [50] contains a pertinent look-up table. Obstacle density is classified into three levels: high, medium, and low. In environments with a high concentration of physical barriers, highly reactive gases are prone to a DDT transition. An example of atmospheric dispersion analysis using ALOHA is given in the next subsection.

5.5.3.2 Example: Syngas Dispersion Modeling in ALOHA

A dispersion analysis from a reference syngas production facility was studied [51]. Two syngas flows were selected as possible limiting safety cases owing to their high-mass flowrates. The first flow is the CO₂-rich syngas downstream of the RWGS reactor while the second flow is the post-selexol syngas that is further topped-up with hydrogen. The physical parameters of these flows are listed in Table 5-11 along with several safety density limits in parts per million (ppm). Protective action criteria (PAC) are concentration levels of chemical materials that threaten or endanger the health and safety of workers or the public. PACs is a collective term that includes acute exposure guideline levels, emergency response planning guidelines, and temporary emergency exposure limits values. Each chemical has its own PAC levels and there are three levels of PAC:

- PAC-1: Mild, transient health effects
- PAC-2: Irreversible or other serious health effects that could impair the ability to take protective action
- PAC-3: Life-threatening health effects.

The maximum credible accident scenario is assumed as a complete rupture at either piping of these syngas flows. Dispersion of the first syngas leakage is modeled using the Gaussian dispersion model while dispersion of the second syngas leakage is modeled using the heavy gas model due to their physical properties.

Table 5-11. Syngas parameters.

Parameter	Syngas #1	Syngas #2
Temperature (°C)	63	211
Pressure (bar)	30	30
Mass flowrate (tons/hr)	214.1	76
Immediately dangerous to life (IDLH, ppm)	4,444.44	1,739.13
PAC-1 (ppm)	277.78	108.70
PAC-2 (ppm)	307.41	120.3
PAC-3 (ppm)	1,222.22	478.3
Emergency Response Planning Guidelines 1 (ERPG-1, ppm)	740.74	289.86
ERPG-2 (ppm)	1,296.30	507.25
ERPG-3 (ppm)	1,851.85	724.64
Lower explosive limit (LEL, %)	27.27	12.20
Upper explosive limit (UEL, %)	46.88	18.99

Two hypothetical reference NPP locations are chosen to demonstrate the proximity of the HTEF and the RWGS reactor to the NPPs. The first site is situated near a river, and the second is positioned in a desert region. A distinctive characteristic of the site by the river is its encirclement by a forest, which acts as a shield for the RWGS reactor, potentially limiting the spread of syngas by interfering with wind patterns, yet also potentially intensifying blast overpressure in the event of a syngas deflagration, as outlined by the BST method. Consequently, there is an interest in examining the dual effects of such an obstacle compared to the unobstructed desert site. Data on the 10-year wind rose graphs for these sites are depicted in Figure 5-14 sourced from the cli-MATE portal of the Midwestern Regional Climate Center [52]. Release scenarios of syngas are modeled in ALOHA for each wind direction to assess the impacts.

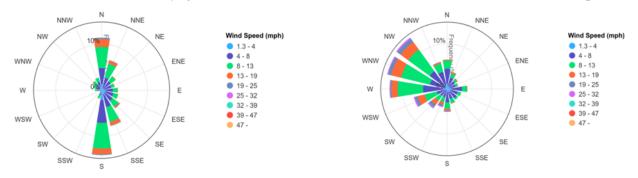


Figure 5-14. 10-year wind speed rose graphs for the reference riverside site (left) and the reference desert site (right).

ALOHA returns threat zone output results for toxic area of vapor clouds, flammable area of vapor clouds, and blast area of vapor cloud explosion for each wind direction. An example output plot is shown in Figure 5-15, where it shows the three Protective Action Criteria (PAC) levels [53] of toxicity boundaries from a release of syngas flows with a 6.2 mph wind from the southwest direction. The top chart is for the first syngas flow modeled using the Gaussian dispersion model, and the bottom chart shows the heavy gas dispersion model for the post-selexol syngas flow. The differences in downwind range and area are likely caused by the extent of advection and the different PAC levels, since the heavy

gas model travels closer to the ground and is therefore less affected by advection until it has diffused significantly to start behaving as a more buoyant gas.

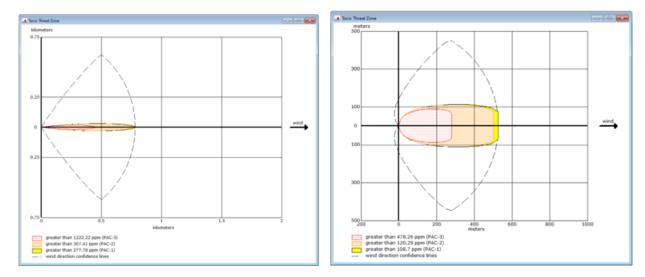


Figure 5-15. ALOHA output window showing syngas toxic areas for a given wind speed and direction, for the Gaussian dispersion model of flow #1 (left) and the heavy gas dispersion model of flow #2 (right)

ALOHA analysis was iterated for all average wind speed and directions in the wind-rose data to obtain toxicity and flammability level of concerns (LOCs) in all directions. The results are mapped in Figure 5-16 for the selected hypothetical sites. For both sites, the LOCs are plotted for the Gaussian dispersion model of flow because it exceeds the distance for the heavy gas dispersion model of flow (Figure 5-15). The figures show a 500 MW_{nom} HTEF located near a PWR. The minimum safe separation distances for hydrogen blast and heat flux damage to PWR SSCs are shown on the map. The figure also shows that toxicity LOCs extend farther than the syngas flammability/detonation LOCs represented by the solid colored lines extending from the syngas facility (blue rectangle). For the riverside site, the dense forest environment causes a "funneling" effect that compresses the ignition wave front such that the resulting overpressure reaches 1 psi from the ignition point. Meanwhile in the open desert site, the overpressure from syngas deflagration is less than 1 psi. Therefore, the LOC is plotted for the low explosive limit (LEL) instead of the 1 psi overpressure. In both cases, results suggest that the RWGS reactor should be placed at greater distance from the NPP relative to the HTEF distance to the NPP. Note however that these are hypothetical sites that do not correspond to any actual operating nuclear plant. Therefore, the results may vary when this methodology is implemented on an actual plant site. Nonetheless, this syngas case study serves as an example to illustrate hazard analysis for neutrally buoyant to heavier than air gases in an integrated energy system.



Figure 5-16. Distances at level of concerns for the hypothetical riverside (left) and desert sites (right).

5.6 Analysis of Heat Flux from Fires and Fireballs

Another hazard arising from explosions is thermal heat flux radiated from fires. The hazards due to heat flux is both from the intensity and exposure time [37]. For that reason, the thermal hazards considered are usually from sustained fires which, in the case of combustible liquids and gases, involve pool fires, jet fires, and fireballs. We reported heat flux calculations from jet hydrogen fires within reference HTEF designs in our previous report [1]. The jet and pool fire scenarios for other industrial facilities are not repeated here because they require data on piping and combustibles flow (e.g., flow rate, pressure, temperature) which is not yet available at the initial research phase. However, an analysis of heat flux resulting from fireballs is presented as an example in this section.

A fireball may arise following a VCE, BLEVE, a boil over, or a pressure vessel burst. Fireball formation occurs as follows: depressurization of a pressurized hot liquid leads to partial flash vaporization and forms a two-phase liquid-vapor mixture. The resulting cloud burns at the edges because the interior concentration is above the flammability limit. Initially semi-spherical and close to the ground, the cloud becomes spherical and rises due to heat and turbulence, which also vaporizes the liquid droplets and reduces the cloud's density. Turbulence aids in efficient combustion, resulting in bright flames and high surface emissive power, allowing radiation to reach far distances. In summary, unique characteristics of fireballs compared to jet and pool fires are that fireballs lift off the ground, grow in size, and radiate an intense thermal flux.

The fireball analysis is selected as an example due to the unique characteristics described above. With its lift-off, its thermal radiation may be less attenuated by surrounding structures compared to ground fire, which may possibly lead to a higher radiation heat flux received by NPP SSCs as illustrated in Figure 5-17. The shortest distance between the center of fireball and the target is denoted as D/2+d, which gives the maximum radiation intensity to the target. Note that although it is called a fireball, it is not always shaped as a ball. Rapid tank failures create approximately spherical fireballs while slower BLEVEs typically create cylindrical fireballs. However, approximating the fireball volume as an equivalent sphere is found to be sufficient in predicting their thermal radiation effects [54].

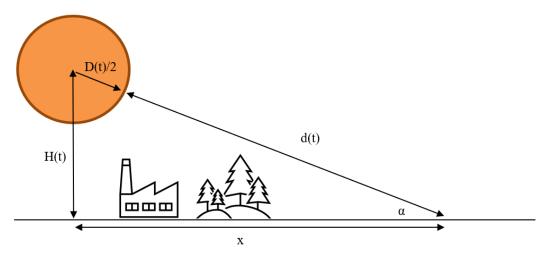


Figure 5-17. Geometry of a fireball and its distance to a target.

A fireball is assumed to form due to an explosion on one of the synfuel and refinery products listed in Table 5-12. It is assumed that there is no domino effect from the explosion. The method to calculate thermal radiation from a fireball is adopted from [35] for a range of products reacted from 1 gallon to 100,000 gallons. The steps to calculate radiative heat flux from a fireball are summarized as follows:

- 1. Assume the ambient pressure at the average atmospheric pressure of 101325 Pa, a relative humidity of 50%, and the ambient temperature at 20°C.
- 2. Calculate fireball's duration as a function of fuel mass, for a range from 1 gallon to 100,000 gallons.
- 3. Calculate fireball's diameter (D) and height (H) as a function of time.
- 4. Calculate the view factor (F), (i.e., the proportion of radiation that strikes the target's surface) as a function of fireball's diameter (D), height (H), and distance to target (d).
- 5. Calculate the atmospheric transmissivity (τ) to account for the atmospheric attenuation of the thermal radiation, as a function of time-varying distance between fireball and target (d).
- 6. Calculate the fireball's emissive power (E), which is the thermal radiation energy emitted omnidirectionally per unit area and time, as a function of time, assuming the radiant heat fraction (ηrad) is 1/3.
- 7. Calculate radiation heat flux for various targets from the initial detonation time until when fireball diminishes. The heat flux to population is multiplied by cosine α as shown in Figure 5-17, assuming most people are standing.
- 8. Calculate thermal dose by integrating the radiation heat flux over time numerically.
- 9. Apply the thermal dose to estimate health effects to population using Eisenberg's probit equations.

Table 5-12. Data of select synfuel products.

Product	Density at 20°C (gr/cm³)	Heat of Combustion (kJ/kg)	Reference
Jet fuel A1 10264	0.78	43.2	[55]
Petroleum naphtha varnish makers & painters	0.75	42.4	[56]
Diesel	0.85	41.36	[57]
Methanol	0.79	19.58	[58]

Results for the jet fuel fireballs are shown in Figure 5-18. As expected, the fireball's duration increases along with the amount of fuel combusted. Plot (a) shows fireball diameter that grows during the first third of its duration and remained constant afterward. Plot (b) shows the fireball's height which increases rapidly during the initial growth period. Both plots suggest the fireball's size and height increases exponentially with the mass of reacted fuel, which agrees with the power formula equation presented in [35]. To compare the heat flux evolution across various fuel masses, an arbitrary distance at 500 meters from the center of fireball was selected in plot (c). It shows the heat flux peaked at the first third of the fireball's duration, which increases exponentially with fuel's mass, then decayed steadily. Plot (d) shows the overall thermal dose throughout the fireball's duration at various separation distances, compared to the thermal dose that can damage equipment and structures of 35–37.5 kW/m² for 30 minutes [59] identified with a dashed red line where applicable. Finally, plots (e) and (f) show the first-degree burn and mortality of the surrounding population at various distances. Although the heat flux is not sufficient to damage structures, it may be harmful to offsite population including nuclear plant personnel who work outdoors such as maintenance crew and physical security guards, if a certain setback distance is not established.

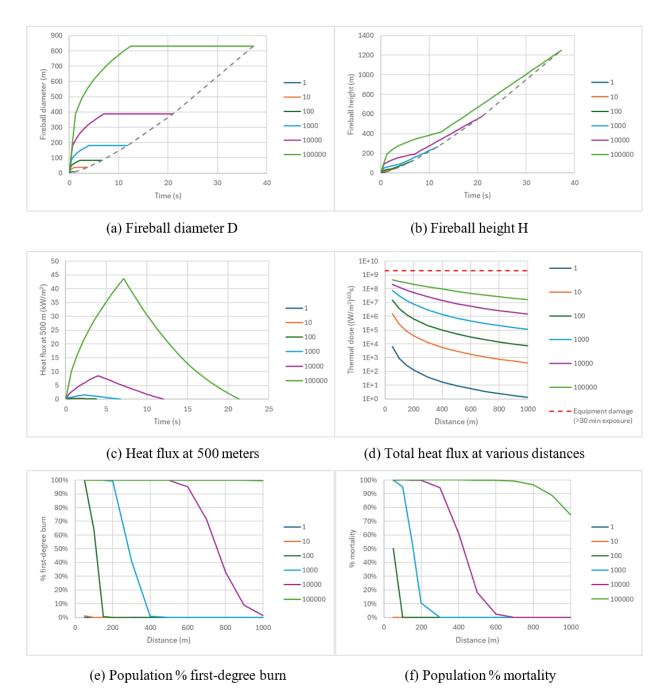


Figure 5-18. Comparison of fireballs resulting from the combustion of 1 gallon to 100,000 gallons of jet fuel.

Figure 5-19 compares the heat fluxes and thermal dose from various synfuel products fireballs, where the heat flux comparison was done at the median of the distance evaluated (i.e., at 500 meters). The figure shows that diesel and naphtha fireballs generate similar outputs of heat flux and thermal dose, meanwhile methanol generate the least thermal output. All three products create less thermal output compared to jet fuel. None of the fireballs radiate heat that can damage nuclear plant SSCs, although a setback distance still needs to be maintained for nuclear plant personnel's safety.

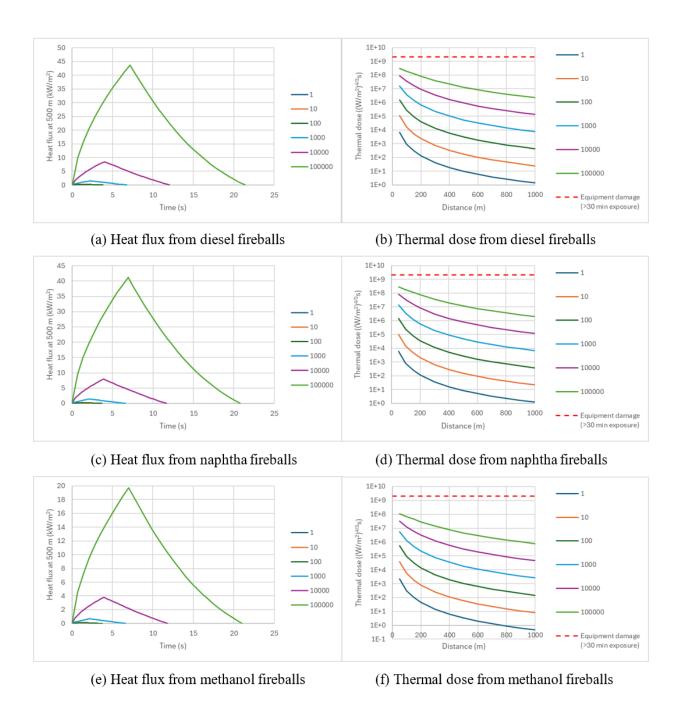


Figure 5-19. Heat flux from various synfuel product fireballs.

5.7 General Plant Transient Due to Overcurrent from Electrical Transmission

The addition of the HES to the NPP requires a direct electrical connection between the NPP and the industrial customer. The design of this connection is described in Section 4.2 and illustrated in Figure 4-8. Most notably, the main turbine generator of the NPP is directly linked to the industrial customer to provide electricity. If there is an overcurrent event at the industrial customer or generator transformer, it could damage the turbine generator if the protections such as circuit breakers fail to isolate the generator.

The turbine generator could also be damaged if the circuit breakers and relay protections fail spuriously and remove the pathway for the load to be dumped.

These protections could also fail if they were to fail due to a seismic event. These seismic considerations were made. The PRA logic includes options for seismic events in five bins ranging from a peak ground acceleration of 0.17 g to 2.12 g. Bin frequencies and gamma uncertainty distribution parameters utilized are from the NRC generic BWR and PWR models. These are reported in Table 5-13.

Extensive searches on seismic fragility constants were performed, and it was not possible to find seismic fragility data for components at as high a level as designed for this transmission system. The fragility constants for the highest voltage components available were used and are reported in Table 5-14. This only records the data used for relays, busbars, and switchgears. The data provided for the busbar was not individual β r and β u but an overall β c [60]. The best data available for circuit breakers and transformers were found in a report that did not explicitly provide fragility constants but provided a fragility curve instead [61]. Values at the seismic bins utilized in this model (Table 5-13) were extracted from the curve and are reported in Table 5-14. It was not possible to find seismic fragility data for components at as high a level as designed for this transmission system, but the data for the highest voltage components available was used.

Table 5-13. Extracted probabilities for high-voltage circuit breakers and transformers [61].

G :	DC(A ()	Probability	
Seismic Bin #	PGA (g)	Circuit Breaker	Transformer
1	0.17	0.020	0.020
2	0.39	0.380	0.380
3	0.71	0.827	0.806
4	1.22	1	0.972
5	2.12	1	1

Table 5-14. Seismic fragility constants used for high-voltage relays, busbars, and switchgear.

C 4T	Fragility Constants			
Component Type	Am (g)	βr	βu	
Relay [61]	0.9	0.35	0.37	
Busbar [60]	1.476	$\beta c = 0.438$		
Switchgear [61]	1.5	0.32	0.48	

5.8 Control Room Habitability

The control room habitability analysis was performed based on the methodology proposed by Ref. [62]. Equation (10) shows the control room transient hazardous gas concentration evolves with time used in Ref. [62].

$$C = C_i (1 - e^{-\frac{V_i}{V}t}) \tag{10}$$

- C_i : Control room hazardous gas intake concentration
- V_i: Control room heating/ventilating/air conditioning (HVAC) intake flow rate (ft³/minute)
- V: Control room volume (ft³)

Equation (10) has two fundamental assumptions:

- 1. The hazardous gas can only travel through the HVAC system from outer area of the building to the control room.
- 2. C_i is independent of time. That is, the hazardous gas concentration does not change with time.

For demonstration of this methodology in the co-located industrial facilities, Syngas would be used and the concentration intake reported in Table 5-11 will be used. Specifically, it is assumed that PAC-1, PAC-2, or PAC-3 reaches to the intake of HVAC system. The control room intake flow rate and the control room volume are shown in Table 5-15.

Table 5-15. Properties of control room air flow.

Parameters	Values	References
V_i	2,220 ft ³ /minute	[62]
V	50,554 ft ³	[62]

Figure 5-20 shows the results of the transient syngas concentration in a control room. It took approximately 120 minutes to reach the maximum concentration. However, due to the fact that the PAC-1, PAC-2, and PAC-3 are all smaller than the IDLH, there is limited concern for operators if the simulated concentration is reached.

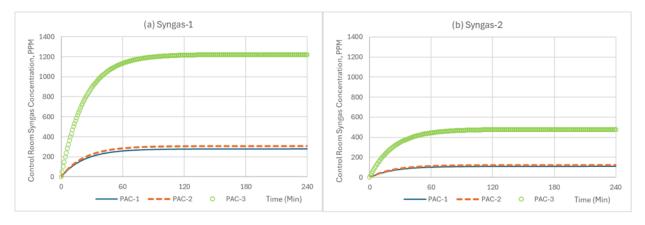


Figure 5-20. Transient syngas concentration in the control room of a nuclear power plant.

Depending on the time last for the transient event, the concentration can be different. Based on the results from Figure 5-20, the maximum concentration can be reached within 2 hours for Syngas-1 and Syngas-2. Syngas-1 has higher PAC-3 concentration compared to Syngas-2. Therefore, 30 minutes, 60 minutes, and 120 minutes of the transient time are used to compare the evolution of the control room syngas concentration as shown in Figure 5-21.

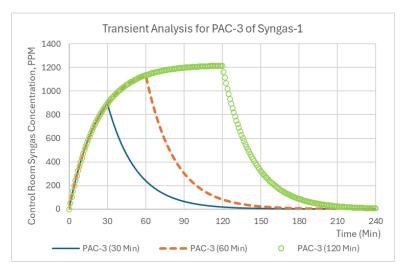


Figure 5-21. Transient syngas concentration in the control room of a nuclear power plant.

In Figure 5-21, it is observed that the earlier the transient can be stopped, the earlier the concentration of the hazardous gas can be restored to the normal value. Note that this analysis does not fully incorporate the ALOHA transient analysis. It was assumed that the leakage of the syngas can be quickly terminated from methanol plant once an accident happened. A full scope of the analysis can be done by integrating the transient analysis from ALOHA starting from the gas leakage in the nearby facility and use the calculated concentration in a shortest distance nearby the HVAC system.

6. PROBABILISTIC RISK ASSESSMENT MODEL

6.1 Electrical Transmission Probabilistic Risk Assessment Model

A PRA model was created to evaluate the probability of a general plant transient due to an overcurrent event damaging the turbine generator as seen at a high level in Figure 4-9. The frequency of this event would add on to the NPP's Transient IE frequency. This could occur four different ways according to the one-line diagram in Figure 4-8: the three-winding transformer at the H2 plant experiences an overcurrent and all circuit breakers fail to trip, the load at the 13.8-kV switchgear pulls too much current and all circuit breakers fail to trip, the generator transformer experiences an overcurrent and the generator circuit breaker (GCB) fails to open and isolate the generator, or a circuit breaker between the industrial customer and generator spuriously trips. For the transformers and circuit breakers between the transformers, the relay protection diagram was used, and the primary and backup relay protection devices were individually accounted for each breaker and transformer. The failure data used for the relays came from the 2020 Industry Average Parameter Estimates which analyzed reactor protection system (RPS) studies data [63]. While this likely refers to low-voltage relays (125 VDC) utilized in RPSs, not high-voltage transmission, this was the best available data. For the switchgear for the industrial facility load, a failure for switchgear rated for over 5 kV was utilized. All other data used were sourced from the Institute of Electrical and Electronics Engineers Gold Book [64].

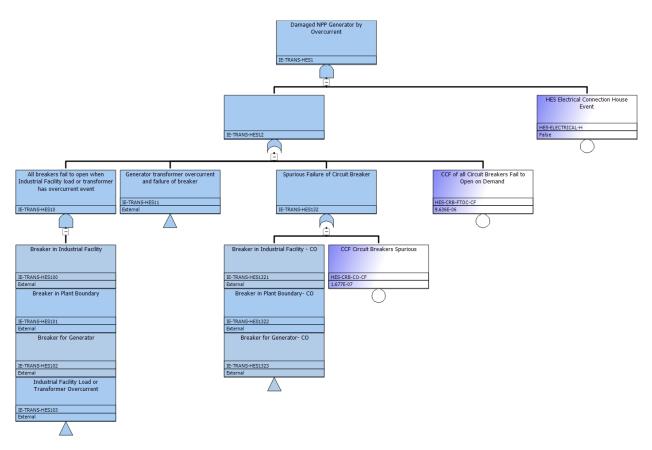


Figure 6-1. Overall FT (IE-TRANS-HES1).

All scenarios were considered in FT IE-TRANS-HES1 (Figure 6-1) indicating either a failure of the generator transformer overcurrent and failure of its breaker, all breakers failing to open when the hydrogen island load or transformer has an overcurrent event will lead to damage of the NPP generator, or a spurious failure of a circuit breaker. All breakers failing scenarios contain the circuit breakers located between the transformers that need to trip to protect the generator from overcurrent in either the transformer or the loads. These scenarios were modeled as a common cause failure (CCF) of all three circuit breakers and sub-FTs in AND gate IE-TRANS-HES10 (Figure 6-1). An application of a primary and backup relay for each breaker and transformer decreases the likelihood of failure along with the presence of the three breakers in series. As long as one of the breakers trips, the generator will be protected. Each of the subtrees representing the logic for the breakers are shown in Figure 6-2, Figure 6-3, and Figure 6-4. The subtree representing overcurrent in either the transformer or the loads is shown in Figure 6-5 with examples of what each A and B branch have in Figure 6-6 and Figure 6-7.

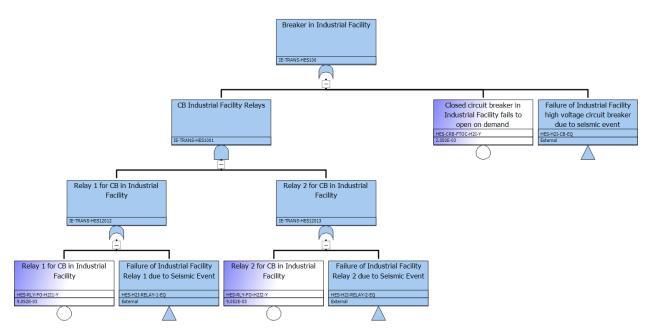


Figure 6-2. Breaker in Industrial Facility (IE-TRANS-HES100).

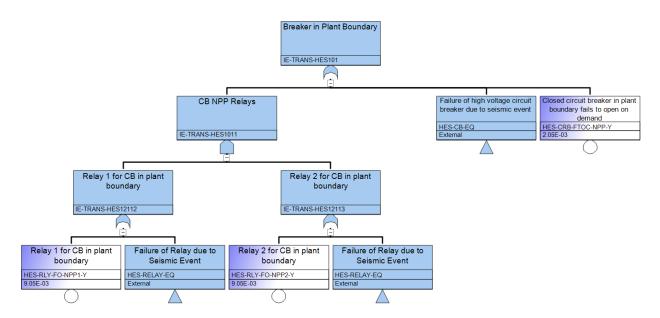


Figure 6-3. Breaker in Plant Boundary (IE-TRANS-HES101).

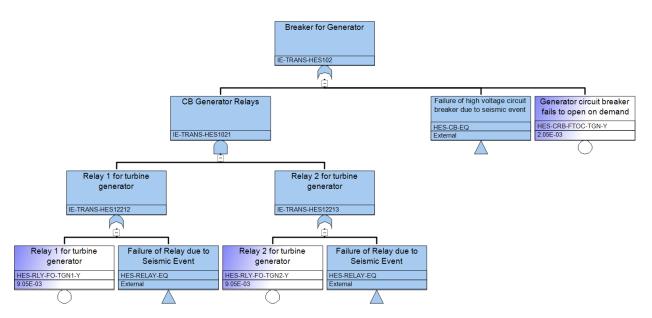


Figure 6-4. Breaker for Generator (IE-TRANS-HES102).

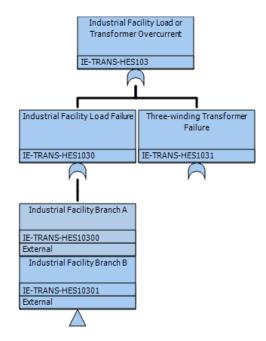


Figure 6-5. Overcurrent by industrial facility transformer or load expanded trees (IE-TRANS-HES103).

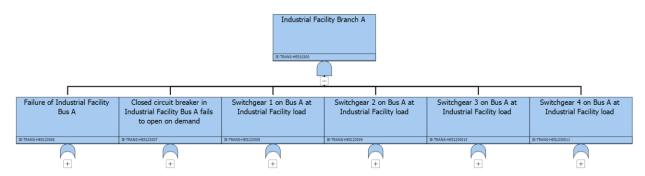


Figure 6-6. Example of a branch in Industrial Facility Load Failure: Branch A.

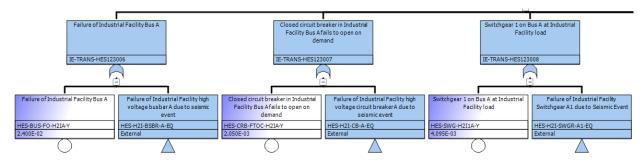


Figure 6-7. Example of Failure of Industrial Facility Bus, Closed circuit breaker in Industrial Facility Bus fails to open on demand, and Switchgear on Bus at Industrial Facility load for Branch A.

The third scenario (Figure 6-8) models the failure of the plant boundary breaker to trip under gate (Figure 6-9) and the occurrence of overcurrent at the generator transformer (Figure 6-10). Since only one circuit breaker separates the transformer from the generator, it is more likely that the generator will be damaged by this scenario. Although, just like the other breakers and transformer, the application of a primary and backup relay for each breaker and transformer decreases the likelihood of failure.

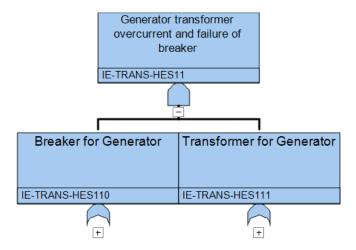


Figure 6-8. Overcurrent by generator step-up transformer.

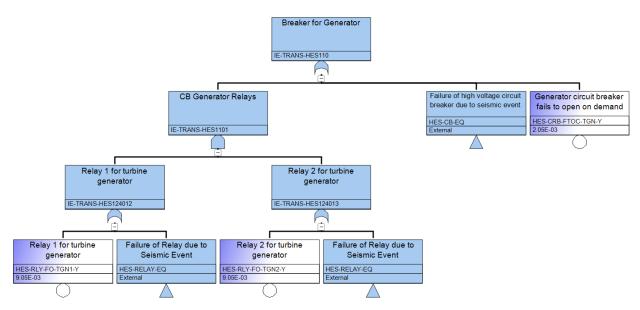


Figure 6-9. Breaker for generator (IE-TRANS-HES110).

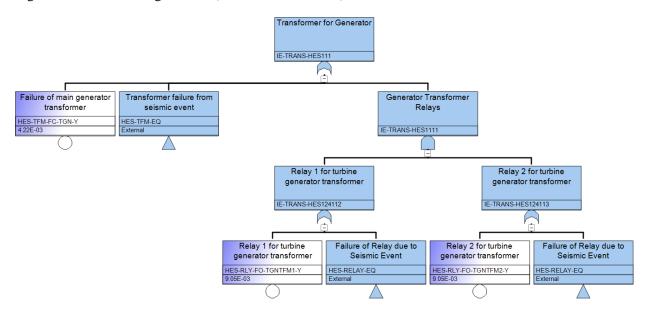


Figure 6-10. Transformer for generator (IE-TRANS-HES111).

The fourth scenario represents the scenario that a circuit breaker spuriously fails, leaving the load from the NPP generator nowhere to go, resulting in a transient at the generator. It is modeled similarly to the first two scenarios in that all breakers between the industrial customer and NPP generator in IE-TRANS-HES132 (Figure 6-1) are referenced.

The primary differences are that the overall connecting logic is an OR gate to represent that only one of the breakers needs to spuriously fail to break the connection (the CCF is minimally two of three breakers) and that the failure of the breaker is not a failure to open on demand, but a spurious failure. An example for the Breaker at Industrial Facility (IE-TRANS-HES1321) is shown in Figure 6-11. While the OR nature of the scenario reduces the failure to a single point for each circuit breaker, spurious failures are multiple orders of magnitude less probable than failures on demand.

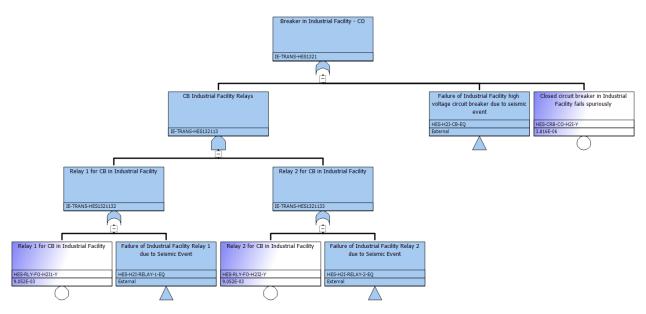


Figure 6-11. Breaker in industrial facility (IE-TRANS-HES1321) as an example for spurious failure of a circuit breaker.

Assuming that the industrial customer can use a similar HES and electrical connection pre-conceptual design with the HTEF as reported in [6], no other scenarios needed to be considered. The report describing the pre-conceptual design [6] states in Section 4.3.5 that "The H₂ production facility is physically and electrically separated from the offsite power circuits. Therefore, there is no impact to offsite power sources or plant safety loads, which normally are powered from offsite power sources." The single line diagram (Figure 4-8) illustrates this further by showing that the offsite power sources are on a different bus than the turbine generator and line to the H₂ production facility in a ring bus arrangement.

6.2 Generic Pressurized Water Reactor Model

The addition of an HES into the steam line creates more venues for the steam to leak out either through pipe breaks or component ruptures. Therefore, one of the possible hazards considered in this study is an increased probability for steam leakage through the new system. In this study, a two-loop generic PWR model is used as a reference. The ET for the Main Steam Line Break initiator is shown in Figure 6-12. A break in the main steam line causes the loss of the ultimate heat sink and therefore the reactor must be tripped. The removal of reactor decay heat depends on whether steam generators are ruptured because of the steam line break. If steam generators are functioning, the auxiliary feedwater (AFW) system supplies feedwater to the steam generators while the main steam/feedwater line is isolated. If the main steam line cannot be isolated, the AFW system cannot inject water due to the high pressure in the line and the high-pressure injection (HPI) is used in its place. In case the AFW system fails, the reactor heat is removed using the feed and bleed mechanism on the primary cooling line. The failure event of steam generators requires mitigation actions as prescribed in the Steam Generator Tube Rupture ET. Meanwhile, the failure of the reactor trip requires mitigation procedures laid out in the Anticipated Transient Without Scram (ATWS) Event Tree. These ETs are provided in Appendix A. Additionally, the existence of an industrial facility near the NPP may create hazards such as detonation, heat flux from fire, and toxicity. These hazards may cause significant damage to critical SSCs. However, it is assumed that the industrial facility is sited at a safe separation distance (Section 5.5) and the risk from these external hazards can be ruled out.

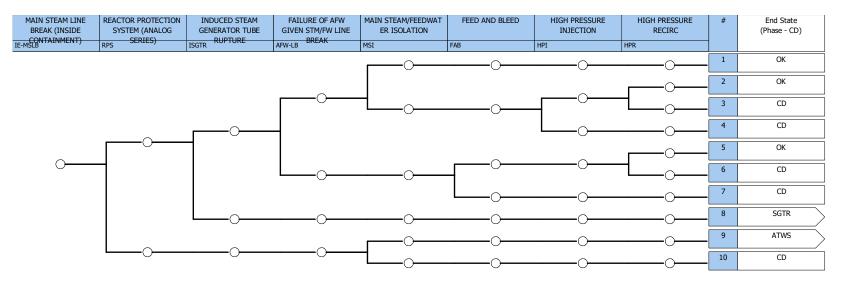


Figure 6-12. MSLB ET (IE-MSLB).

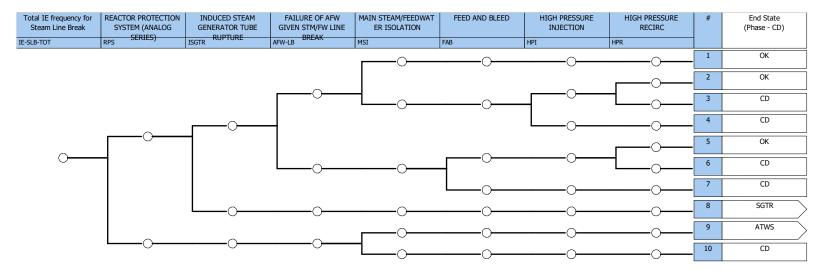


Figure 6-13. MSLB ET with HES (IE-SLB-TOT).

6.2.1 Heat Extraction System Linkage into the Pressurized-water Reactor Model

The addition of the HES that taps into the main steam line of an NPP creates additional points where steam may leak out of the secondary cooling loop. The additional frequency from HES is added to the existing base IE frequency of the steam line break ET using an IE FT as shown in Figure 6-14. The IE FT developed for the 100 MW_{nom} (\leq 100 MW_t) industrial customer design in Figure 4-2 is shown in Figure 6-15. The FT developed for the 500 MW_{nom} (101-200 MW_t) design in Figure 4-4 is shown in Figure 6-16, and the FT developed for the 1000 MW_{nom} (> 200 MW_t) design in Figure 4-6 is shown in Figure 6-17. The thermal requirements for each design are shown in Table 3-1. The top events of these trees add up to the total steam line break IE frequency, which is used as the initiator for the new steam line break ET as shown in Figure 6-13. Figure 6-13 House events are used to select which industrial customer size is to be used in the FT quantification.

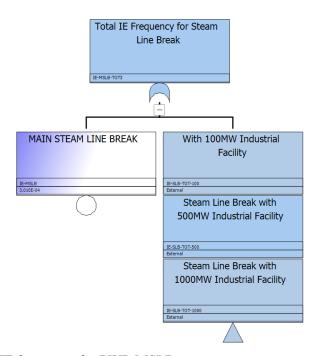


Figure 6-14. FT for total IE frequency for PWR MSLB.

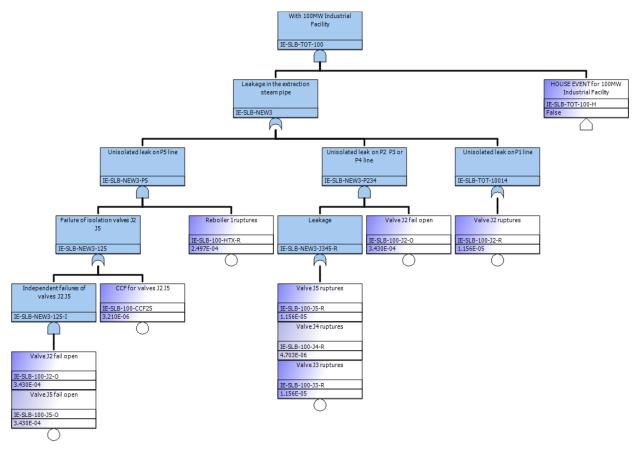


Figure 6-15. FT for Total Initiating Event frequency for MSLB with 100 MW industrial customer.

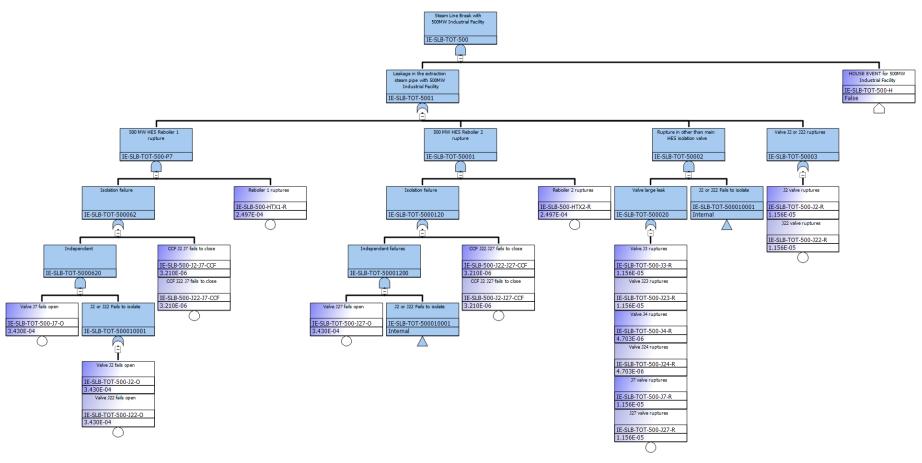


Figure 6-16. FT for Total Initiating Event frequency for MSLB with 500 MW industrial customer.

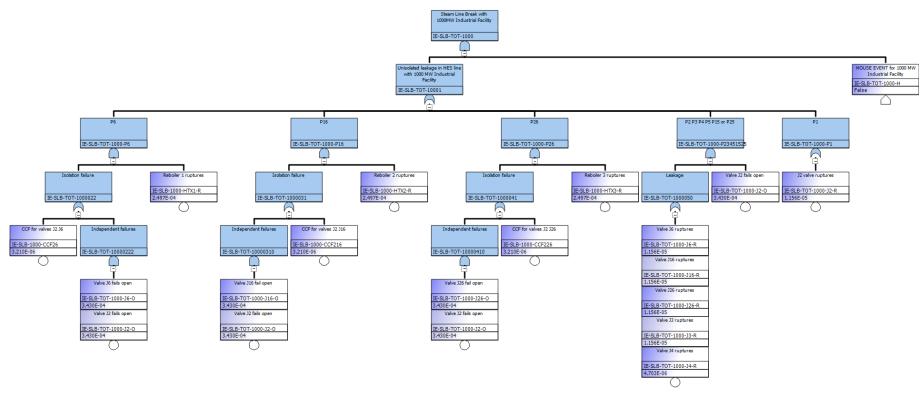


Figure 6-17. FT for Total Initiating Event frequency for MSLB with 1000 MW industrial customer.

6.3 Generic Boiling Water Reactor Model

Similar to the PWR, the HES in the BWR taps steam from the main steam line. A loss of the steam flow rate due to a leakage event in the HES may lead to a general transient event. The mitigation procedure for this event is shown in Figure 6-18. The transient can be mitigated safely if reactor power generation is shut down, the offsite power is available, the safety relief valves remain closed to preserve coolant inventory, and the power conversion system is running. If this power conversion system fails, the HPI system is activated followed by suppression pool cooling. Without the automatic suppression pool cooling, operators need to depressurize the reactor manually and perform the control rod drive injection. Further mitigation sequences can be deducted from the figure, in which various redundant measures are available including a low-pressure injection (LPI) system, shutdown cooling, containment spray, and containment venting.

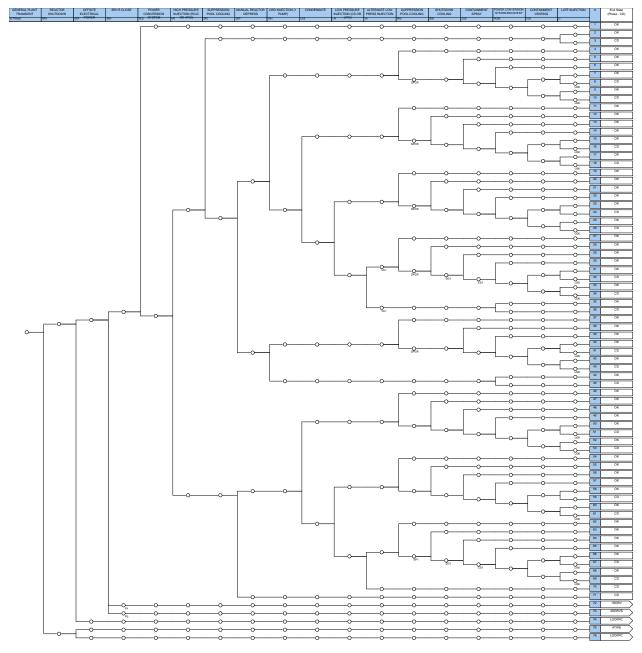


Figure 6-18. General Transient ET (IE-TRANS).

6.3.1 Heat Extraction System Linkage into the Boiling Water Reactor Model

The mitigation procedure for a steam line break in the HES is shown in Figure 6-19. When the event occurs, the core will be damaged if the RPS fails or if the MSIVs fail to close. If both systems function properly, the mitigation tree transfers to the General Transient event tree as shown in Figure 6-18. However, since the General Transient tree is used as is, there needs to be a set of linkage rules to customize the tree based on the initiator (i.e., a steam line break in the HES). These linkage rules are set as pictured in Figure 6-20. It instructs SAPHIRE (Systems Analysis Programs for Hands-on Integrated Reliability Evaluations) to activate the LSSB-HES Flag Set when the initiator is IE-LSSB-HES. This instruction is also carried over to the transfer ETs (i.e., General Transient). The LSSB-HES Flag Set is set

up as shown in Figure 6-21. It activates the HE-SLB-TOT house event and changes its state from False to True. The same logic is used for other HES designs.

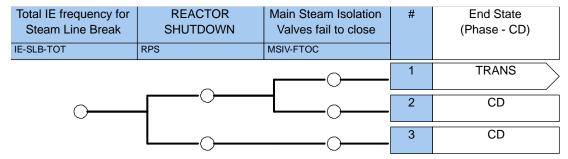


Figure 6-19. Initiating event for steam line break in the HES (IE-SLB-TOT).

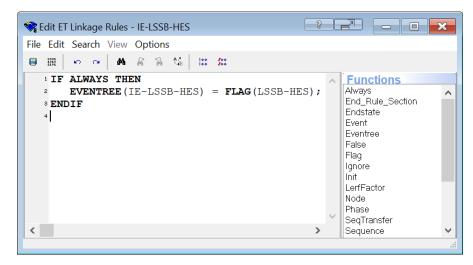


Figure 6-20. Linkage rules for the IE-LSSB-HES ET

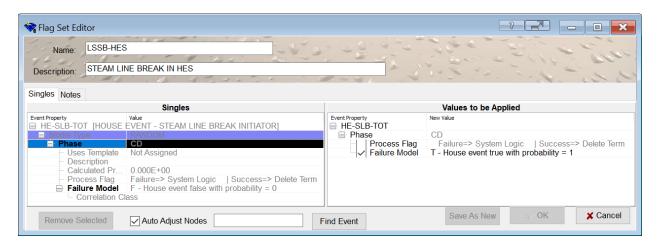


Figure 6-21. LSSB-HES flag editor.

As indicated in Figure 6-18, the IE-SLB-TOT ET transitions to the TRANS tree only when RPS functions successfully. For that reason, the RPS top event in the TRANS tree should not be evaluated

again when the sequence originates from IE-SLB-TOT FT that determines the steam line break IE frequency. This logic is made possible by adding a complement of HE-SLB-TOT as shown in the RPS FT (Figure 6-22). This event is coupled in an AND gate with the other events that may cause RPS to fail. With this configuration, when the IE-SLB-TOT ET transitions to the TRANS tree, the LSSB-HES flag is activated, and the HE-SLB-TOT house event is set to true. Therefore, its complement becomes false, and the RPS failure top event does not occur. Meanwhile, when the TRANS tree is activated after the MSIV is closed, the Power Conversion System (PCS) is always off. This logic is implemented by adding the HE-SLB-TOT house event in an OR gate to the PCS and PCS recovery FT, as shown in Figure 6-23 and Figure 6-24 respectively.

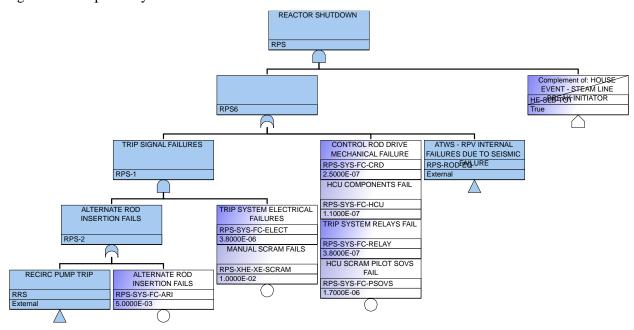


Figure 6-22. RPS FT.

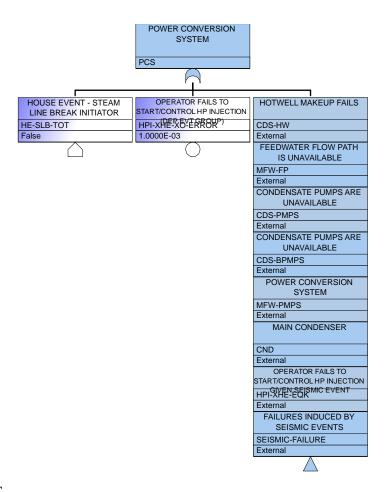


Figure 6-23. PCS FT.

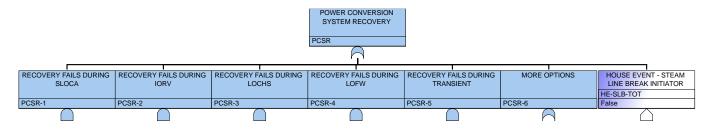


Figure 6-24. PCSR FT.

7. RESULTS OF PROBABILISTIC RISK ASSESSMENT

7.1 Nominal Probabilistic Risk Assessment Results

The required modifications listed in this report to support an industrial customer affect the DBA IE frequencies and the CDF contribution of the Main Steam Line Break and General Transient. The results of the deterministic hazards analyses that set design assumptions for the PRA are provided throughout this section by nuclear reactor type and topic.

7.1.1 Pressurized-water Reactor Probabilistic Risk Assessment Results

This section reports the IE frequencies and CDF for the nominal generic PWR model and the increases resulting from the addition of the 100, 500, and 1000 MW_{nom} industrial customers (i.e., \leq 100, 100–200, and >200 MW_t respectively) and the electrical connection to the industrial customers.

The overall PWR CDF increased minimally across the three industrial customer HES designs (Table 7-1). The significance of the overall CDF increase is for RG 1.174 licensing support, if desired.

Table 7-1.	Overall PWR	core damage	frequency	results by	v HES	modification.

PWR Modification State	Overall CDF	% increase
Nominal	6.54E-06	nom
$\leq 100 \text{ MW}_{t} \text{ HES}$	6.55E-06	0.15%
101–200 MW _t HES	6.56E-06	0.31%
> 200 MW _t HES	6.55E-06	0.15%

The HES design is the driver in the IE and CDF results for the steam line break DBA (Table 7-2). The most significant component in the HES designs are the motor operated isolation valves. The rupture failure of the isolation valves would require the NPP to shut down and the MSIVs to close to prevent loss of primary coolant, regardless of where the steam is tapped. The frequency of rupture of this motor operated valve is 1.2E-05/year. The reboiler rupture failure is logically ANDed with the failure of the isolation valves to close, so the unisolated failure probability for the reboiler ruptures is 8.3E-10. This is five orders of magnitude below the isolation valve rupture which is why the 1000 MW_{nom} industrial customer HES design, with three reboilers, has the same overall IE and CDF as the 100 MW_{nom} industrial customer HES design with only one reboiler. The 500 MW_{nom} industrial customer HES design shows the higher increase in IE and CDF because of the two isolation valves for the two steam taps.

Table 7-2. PWR Steam line break results by HES modification

PWR Modification State	Steam Line Break IE Frequency	% Increase	Steam Line Break CDF	% Increase
Nominal	3.01E-04	nom	2.51E-07	nom
≤ 100 MW _t HES	3.13E-04	3.85%	2.60E-07	3.83%
101–200 MW _t HES	3.24E-04	7.69%	2.70E-07	7.70%
> 200 MW _t HES	3.13E-04	3.86%	2.604E-07	3.83%

The event that can increase the Transient IE is the overcurrent failure of the electrical connection between the NPP generator and an HTEF or industrial user. The design was originally for an HTEF so these results are a baseline under the assumption that this design would work for other industrial users. More extensive designs may modify these results and would need to be listed separately. For this design, the increase in the Transient IE is 2.76E-4, three orders of magnitude below the nominal Transient IE and is the same for each HTEF. Note that the percent increase is not significant enough to show in Table 7-3.

Table 7-3. PWR Transient results by HES modification

PWR Modification State	Transient IE Frequency	% Increase	Transient CDF	% Increase
Nominal	6.76E-01	nom	2.01E-07	nom
$\leq 100 \text{ MW}_{\text{t}} \text{ HES}$	6.76E-01	0.00%	2.01E-07	0.00%
101–200 MW _t HES	6.76E-01	0.00%	2.01E-07	0.00%
> 200 MW _t HES	6.76E-01	0.00%	2.01E-07	0.00%

Seismic analysis results are from the increased failure probabilities of the components involved and obviously do not affect the IE frequencies of the seismic events. The summation of all seismic bins for the PRA model by HES modification show that the electrical seismic event additions do not increase the seismic CDF significantly as shown in Table 7-4. Again, this design was originally for an HTEF so these results are a baseline under the assumption that this design would work for other industrial users. More extensive designs may modify these results and would need to be listed separately.

Table 7-4. PWR Overall seismic results by HES modification

PWR Modification State	Seismic CDF	% Increase
Nominal	3.56E-06	nom
$\leq 100 \text{ MW}_{\text{t}} \text{ HES}$	3.56E-06	0.00122%
101–200 MW _t HES	3.56E-06	0.00122%
> 200 MW _t HES	3.56E-06	0.00122%

7.1.2 Boiling Water Reactor Probabilistic Risk Assessment Results

This section reports the IE frequencies and CDF for the nominal generic BWR model and the increases resulting from the addition of the 100, 500, and 1000 MW_{nom} industrial customers and the electrical connection to the industrial customers.

The overall BWR CDF increased minimally across the three industrial customer HES designs (Table 7-5). The significance of the overall CDF increase is for RG 1.174 licensing support, if desired. The very low changes in BWR CDF are due to the higher starting point of the nominal CDF and the same probabilistic results of the HES additions as are used in the PWR.

Table 7-5. Overall BWR core damage frequency results by HES modification

BWR Modification State	Overall CDF	% increase
Nominal	2.55E-05	nom
$\leq 100 \text{ MW}_{t} \text{ HES}$	2.55E-05	0.00016%
101–200 MW _t HES	2.55E-05	0.00018%
> 200 MW _t HES	2.55E-05	0.00016%

The HES design is the driver in the IE and CDF results for the steam line break DBA (Table 7-6). The most significant component in the HES designs are the motor operated isolation valves. The rupture failure of the isolation valves would require the NPP to shut down and the MSIVs to close to prevent loss of primary coolant, regardless of where the steam is tapped. The frequency of rupture of this motor operated valve is 1.2E-05 /y. The reboiler rupture failure is logically ANDed with the failure of the isolation valves to close, so the unisolated failure probability for the reboiler ruptures is 8.3E-10. This is five orders of magnitude below the isolation valve rupture which is why the 1000 MW_{nom} industrial customer HES design, with three reboilers, has the same overall IE and CDF as the 100 MW_{nom} industrial customer HES design with only one reboiler. The 500 MW_{nom} industrial customer HES design shows the higher increase in IE and CDF because of the two isolation valves for the two steam taps.

Table 7-6. BWR Steam line break results by HES modification

BWR Modification State	Steam Line Break IE Frequency	% Increase	Steam Line Break CDF	% Increase
Nominal	2.53E-03	nom	1.23E-07	nom
$\leq 100 \text{ MW}_{\text{t}} \text{ HES}$	2.54E-03	0.47%	1.24E-07	0.49%
101–200 MW _t HES	2.55E-03	0.91%	1.24E-07	0.89%
> 200 MW _t HES	2.54E-03	0.47%	1.24E-07	0.49%

The event that can increase the Transient IE is the overcurrent failure of the electrical connection between the NPP generator and the industrial customer. The increase in the Transient IE is 2.76E-4, three orders of magnitude below the nominal Transient IE and is the same for each industrial customer. Note that the percent increase is not significant enough to show in Table 7-7.

Table 7-7. BWR Transient results by HES modification

BWR Modification State	Transient IE Frequency	% Increase	Transient CDF	% Increase
Nominal	7.40E-01	nom	3.88E-06	nom
$\leq 100 \text{ MW}_{\text{t}} \text{ HES}$	7.40E-01	0.00%	3.88E-06	0.00%
101–200 MW _t HES	7.40E-01	0.00%	3.88E-06	0.00%
> 200 MW _t HES	7.40E-01	0.00%	3.88E-06	0.00%

Seismic analysis results are from the increased failure probabilities of the components involved and obviously do not affect the IE frequencies of the seismic events. The summation of all seismic bins for the PRA model by HES modification show that the electrical seismic event additions do not increase the seismic CDF significantly as shown in Table 7-8.

Table 7-8. BWR Overall seismic results by HES modification

BWR Modification State	Seismic CDF	% Increase
Nominal	3.56E-06	nom
$\leq 100 \text{ MW}_{\text{t}} \text{ HES}$	3.56E-06	0.00122%
101–200 MW _t HES	3.56E-06	0.00122%
> 200 MW _t HES	3.56E-06	0.00122%

8. LICENSING PATHWAY SUPPORT FROM PROBABILISTIC RISK ASSESSMENT

The NRC uses Codes of Federal Regulations and develops various regulatory guides to assist license applicants' implementation of NRC regulations by providing evaluation techniques and data used by the NRC staff. Two distinct pathways through guides and Codes of Federal Regulations are used in the proposed LWR plant configuration change approval.

One pathway uses 10 CFR 50.59 [4] to review the effects of the proposed small changes to the NPP, including minimal increases in frequencies of DBAs, amending the updated final safety analysis report, and determining whether an LAR is required. This pathway is dependent on the IE frequencies determination, which is on the front end of the PRA.

While the 10 CFR 50.59 evaluation does not specifically require a PRA, the PRA does provide numerical evidence of the effect of the proposed activities.

A supporting pathway utilizes RG 1.174 [5] using risk-informed metrics to approve a plant configuration change based on the effect on the overall CDF and LERF of an approved PRA. This pathway is dependent on the tail end of the analysis, the CDF and LERF resulting metrics of the PRA.

The final pathway is the LAR process, which would utilize PRA results as well; however, the process utilizes 10 CFR 50.90, "Application for amendment of license or construction permit at request of holder" [65] and is historically a lengthier review and monetary burden.

8.1 Licensing Process through 10 CFR 50.59

This licensing pathway first uses 10 CFR 50.59 [4] to determine if an LAR would be required via 10 CFR 50.90 [65]. Changes that meet the 10 CFR 50.59 requirements do not require additional NRC review and approval. In a studies commissioned by LWRS [66][67], the effects on DBAs of a PWR with the addition of an HES were evaluated for adherence to the following eight criteria:

- 1. Result in more than a minimal increase in the frequency of occurrence of an accident previously evaluated in the final safety analysis report (as updated)
- 2. Result in more than a minimal increase in the likelihood of occurrence of a malfunction of a structure, system, or component important to safety previously evaluated in the final safety analysis report (as updated)
- 3. Result in more than a minimal increase in the consequences of an accident previously evaluated in the final safety analysis report (as updated)
- 4. Result in more than a minimal increase in the consequences of a malfunction of an SSC important to safety previously evaluated in the final safety analysis report (as updated)
- 5. Create a possibility for an accident of a different type than any previously evaluated in the final safety analysis report (as updated)
- 6. Create a possibility for a malfunction of an SSC important to safety with a different result than any previously evaluated in the final safety analysis report (as updated)
- 7. Result in a design basis limit for a fission product barrier as described in the final safety analysis report (as updated) being exceeded or altered
- 8. Result in a departure from a method of evaluation described in the final safety analysis report (as updated) used in establishing the design bases or in the safety analyses.

If the above criteria are not met, the 10 CFR 50.59 process cannot be used to implement the plant modification, and an LAR must be submitted to the NRC for review and approval.

The S&L study noted that all deterministic criteria are met for a 10 CFR 50.59 application based on the modifications noted in their report [67]. This report uses the same modifications as the S&L study for the HESs to support 100 and 500 MW_{nom} industrial customer designs. The HES for the 1000 MW_{nom} industrial customer is an extension design proposed by INL based on generalized recommendations from S&L and other LWR experts. As noted in References [66] and [67], nearly all criteria are readily met for a modification such as the HES, but there was not enough data available at the time to determine whether item 1 (minimal increase in DBA frequency) is met probabilistically. A minimal increase is traditionally understood to be \leq 10% as proposed by the Nuclear Energy Institute (NEI), "Guidelines for 10 CFR 50.59 Implementation," [68]. Specifically, Example 8 states:

The change in likelihood of occurrence of a malfunction is calculated in support of the evaluation and increases by more than a factor of two. Note: The factor of two should be applied at the component level. Certain changes that satisfy the factor of two limit on increasing likelihood of occurrence of malfunction may meet one of the other criteria for requiring prior NRC approval, e.g., exceed the minimal increase standard for accident/transient frequency under criterion 10 CFR 50.59(c)(2)(i). For example, a change that increases the likelihood of malfunction of an emergency diesel generator by a factor of two may cause more than a 10% increase in the frequency of station blackout.

Reference [68] is endorsed by the NRC in "Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments," Regulatory Guide (RG) 1.187 [69]. This PRA found the largest increase in a DBA yearly IE frequency to be 7.69% (Large Steam Line Break for the PWR) from all considered HES designs, thus meeting the item 1 criteria for 10 CFR 50.59.

8.2 Adherence to the Site Fire Protection Plan

The placement of an industrial facility within the OCA of the existing NPP site will be within the NRC's regulatory jurisdiction. This means that, among other things, the safe siting separation distance will be dictated by the site's existing fire protection program/plan (FPP). As of the publication of this report there are only hydrogen facilities proposed for placement within the OCA. A report on code and licensing separation distance considerations [70], prepared concurrently with this report, covers FPP adherence for a hydrogen facility co-located with an NPP.

All sites have the FPP within the final safety analysis report and each site used a plan agreed to for their license. This can include a deterministic approach through 10 CFR 50 Appendix R and/or a risk-informed approach through 10 CFR 50.48(c) [71]. The risk-informed approach is also known as "NFPA-805 plants" which is the NFPA code called out within the 10 CFR 50.48.

Independent of the FPP classification, the first step in self-evaluating a co-located industrial facility is to clearly define the change and conduct an impact review. If the change is determined to be minor, documenting the change and basis is sufficient. Next a preliminary risk screen is performed to evaluate if the impact is potentially more than minimal for NFPA 805 licensed plants. The results of this screen determine if a qualitative risk evaluation is sufficient, or a more detailed quantitative evaluation is necessary. Once the risk evaluation is completed, the results are compared against the delta CDF and LERF acceptance criteria (Section 8.3). Assuming the risk acceptance criteria are met, safe separation distance is determined by strategies such as detailed in Section 5.5.

An HTEF or other industrial facility located in an NPP OCA today would likely be designed to NFPA 2 [17]. As described within NFPA 2, the intent "shall be to provide fundamental safeguards for the generation, installation, storage, piping, use, and handling of hydrogen in compressed gas (GH2) form or cryogenic liquid (LH2) form." Thus, the general associated piping and equipment and other code safety standards to be employed for the HTEF as a stand-alone compressed hydrogen gas facility in the NPP OCA currently would be expected to meet NFPA 2 although this code is not directly referenced within the licensing pedigree of either Appendix R or NFPA 805 plants. Employing this widely accepted code standard would however be wisely included as a design evaluation basis provided under the fire protection engineering evaluation allowed by both Appendix R and NFPA 805 plant self-evaluation processes for the change to the facility associated with co-location of an HTEF or other industrial facility.

More information can be found in Ref. [70], however, as documented in both the heat flux methodology and criterion of 37.5 kW/m^2 for a 30 minute duration and the blast overpressure methodology and criterion limit of ≤ 1.0 psi set forth in this report provide conservative safe separation distances when compared to any of the codes and regulations listed in this section.

8.3 Licensing Support through RG 1.174

RG 1.174 [5] provides general guidance concerning analysis of the risk associated with proposed changes in plant design and operation. Specifically, thresholds and guidelines are provided for comparison with Level 1 PRA results for CDF and LERF.

As seen in Figure 8-1, CDF should be below \sim 1E-3/y overall and the change in overall CDF should be below a magnitude of 1E-5/y. Any plant that starts at a 1E-4 or more CDF requires less than 1E-6/y increase in CDF to be considered. For the addition of HES and direct electrical connection, both the generic BWR and PWR nominal CDFs are below 1E-5/y. The largest increase in CDF of the two LWRs in this report is Δ CDF of 2.0E-8/y for the generic PWR with a 500 MW_{nom} HES design. This result is well within these metrics; therefore, the NRC most likely considers this a small change consistent with the intent of the Commission's Safety Goal Policy Statement [72] and a detailed quantitative assessment of the base values of CDF is not necessary for the license review.

If the above criteria for CDF were not met, an LAR must be submitted to the NRC for review and approval.

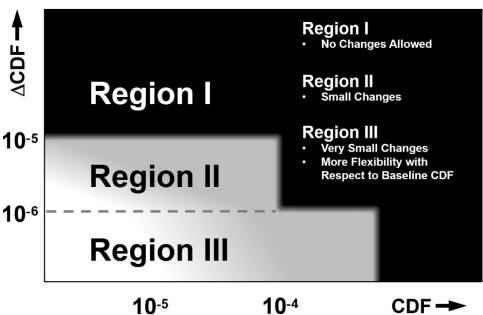


Figure 8-1. Acceptance guidelines for CDF.

As seen in Figure 8-2, LERF should be below \sim 1E-4 overall and the change in overall LERF should be below a magnitude of 1E-6. Both the generic BWR and PWR nominal LERFs are below 1E-6/y. The largest increase in LERF of the two LWRs in this report is a Δ LERF of 5.1E-7/y for the generic BWR with a 500 MW_{nom} HES design. This result is well within these metrics; therefore, the NRC most likely considers this a small change consistent with the intent of the Commission's Safety Goal Policy Statement [72] and a detailed quantitative assessment of the base values of CDF is not necessary for the license review. The LERF for these models is well within Region III.

If the above criteria for LERF were not met, an LAR must be submitted to the NRC for review and approval.

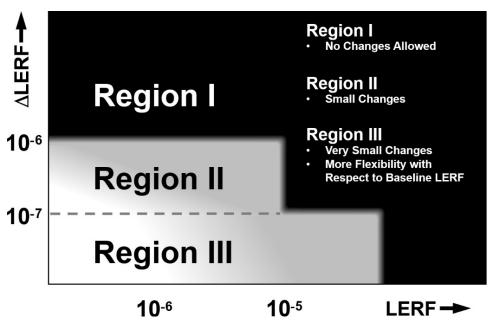


Figure 8-2. Acceptance guidelines for LERF.

8.4 Licensing Support through RG 1.91

RG 1.91 [3] is the current NRC Regulation Guide for evaluating explosion risks near an NPP, meaning outside of the OCA. Some NPPs have used RG 1.91 analyses in their safety case. The TNT mass equivalent methodology is used, and standoff distances are required to limit a maximum credible accident to less than a 1 psi overpressure. We recommend, along with reference [70], that licensees use the RG 1.91 methodology and criteria with a maximum break-type leak as a bounding overpressure effects tool for establishing a safe separation distance between the industrial facility and the NPP SSCs to provide a conservative assessment of safe separation distance when compared to NFPA minimum standards. The NFPA standards allow lesser experientially-based leakage sizes, but the decrease in safe siting distance is generally not advantageous given typical NPP site configurations and the longer distances inherent between NPP SSC's and logical siting locations within or outside of the OCA.

8.5 Licensing Amendment Review Process

Should the prior processes fail to approve a change in the LWR, the last resort would be a detailed request for an LAR. As stated in Reference [66]:

10 CFR 50.90 is the governing regulation for the process undertaken by the licensee to develop and submit an LAR. This regulation states that the application fully describes the changes desired and is to follow the form prescribed for the original updated final safety analysis report submittal. An LAR is required when a change to the technical specifications is desired for whatever purpose. The LAR is developed by the licensee staff and is reviewed by internal committees and management to ensure that the technical content is correct and meets management approval.

The NRC LAR review is extensive and typically involves meetings with the licensee and the opportunity for public meetings per 10 CFR 50.91, "Notice for Public Comment; State Consultation" [73]. The NRC issues requests for additional information to obtain responses from the licensee as a result of the NRC review. 19 CFR 50.92, "Issuance of Amendment" [74] includes a "no significant hazards" consideration to determine whether any of the following conditions exist based on the NRC LAR review:

- Involves a significant increase in the probability or consequences of a previously evaluated accident
- Creates the possibility of a new of different kind of accident from any previously evaluated accident
- Involves a significant reduction in margin of safety.

Provided these regulatory requirements are met, the NRC issues a safety evaluation that approves the LAR including the technical specification revisions.

9. CONCLUSIONS

Generic specifications of industrial customers were used in this report and some processes to produce carbon-reduced fuels were used in the pre-conceptual design stage. The reference facilities were a methanol plant, a syngas production, an oil refinery, and a wood pulp and paper mill. Hazards were identified and assessed for potential consequences through accidentology and FMEAs. The safety of the NPP was the primary focus of this research; however, FMEAs were expanded to include informational results for the industrial facilities, public safety and perception, and economic concerns. Methodologies were presented for determining safe separation distances from these potential hazards that ensured the safety of the NPP, workers, and the public. Probabilistic risk results were presented for the changes required of the NPP to support industrial customers.

It is important to eliminate, through distance and/or mitigation, the external hazards presented by the reference industrial facilities through safe separation distance of the facility to the nearest NPP SSC. Deterministic analyses, approaches, and considerations presented in this report can be used as a part of an overall strategy to define this safe separation distance between the point of hazards presented by the industrial customer to the nearest NPP SSC. This safe separation distance is used to meet FPP criteria set forth in NFPA standards, U.S. Codes of Federal Regulations, and regulation guidance that the NRC uses to license NPPs and allow the self-assessment of NPP utilities to make small changes to their power plant. Engineered safety barriers can also be credited while following safety codes and regulations. Beyond licensing requirements for NPP safety, the report's example deterministic analyses provide tools to evaluate the safety of workers and the public near industrial facilities in case of fire, detonation, and toxicity.

A PRA was modeled and performed on the required additions to the NPP for the thermal extraction systems and direct electrical connection to the industrial customer. The PRA results are all below the IE frequency increases considered minimal for 10 CFR 50.59 changes.

The hazards analyses and PRAs presented in this report provide a toolbox and starting point for site-specific assessments that can be used to ensure the safety of the NPP, the industrial facility for some discrete failures, and the public and to help meet regulatory licensing criteria to co-locate an industrial facility near an NPP. No attempt was made to meet any federal regulatory requirements or safety standards, or to assess all hazards present for the industrial facilities themselves beyond the assessment of hazards identified as potential threats to NPP safety.

10. REFERENCES

- [1] Vedros, K. G., R. Christian, C. Otani. 2024. "Hazards and Probabilistic Risk Assessments of a Light-Water Reactor Coupled with Electrolysis Hydrogen Production Plants." Report No. INL/RPT-23-74319, Revision 1, Idaho Falls, Idaho: Idaho National Laboratory.
- [2] Joseck, F., et al. 2024. "Thermal Integration of Advanced Nuclear Reactors with a Reference Refinery, Methanol Synthesis, and Wood Pulp Plant." Report No. INL/RPT-23-03951, Idaho Falls, Idaho: Idaho National Laboratory.
- [3] U.S. Nuclear Regulatory Commission. 2013. Evaluations of Explosions Postulated to Occur at Nearby Facilities and on Transportation Routes Near Nuclear Power Plants. RG-1.91, Revision 3, U.S. Nuclear Regulatory Commission.
- [4] U.S. Nuclear Regulatory Commission. 2017. "10 CFR 50.59 Changes, Tests and Experiments." U.S. Code of Federal Regulations, https://www.nrc.gov/reading-rm/doc-collections/cfr/part050/part050-0059.html
- [5] NRC. 2018. "An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis," RG-1.174, Revision 3, U.S. Nuclear Regulatory Commission, https://www.nrc.gov/docs/ML1731/ML17317A256.pdf
- [6] Sargent & Lundy. 2022. "Nuclear Power Plant Pre-Conceptual Design Support for Large-Scale Hydrogen Production Facility," Report No. SL-016181, Revision 1, November 18, 2022.
- [7] Cappello, V., H. Delgado, P. Sun, A. Elgowainy. 2023. "Synfuel production process with methanol as intermediate product: Aspen Plus models of synfuel production integrated with nuclear power plant at different scales." Report No. ANL/ESIA-23/3, Lemont, Illinois: Argonne National Laboratory.
- [8] Stolecka, K. and A. Rusin. 2020. "Analysis of hazards related to syngas production and transport," Renewable Energy, Vol. 146, pp. 2535-2555, February 2020. https://doi.org/10.1016/j.renene.2019.08.102
- [9] Dolan, G. A. 2020. "Methanol: Emerging Global Energy Markets." Presented at 16th Annual State of the Energy Industry Forum, Washington, DC, January 2020. Available: https://usea.org/sites/default/files/event-/SOTEI%202020_Greg%20Dolan_MI%20USEA%20Annual%20Forum%2023%20Jan%202020.pdf
- [10] Food and Agriculture Organization of the United Nations. 2023. "Pulp and Paper Capacities: Survey 2022-2024." Rome, Italy, 2023.
- [11] Hancock, S., T. Westover, Y. Luo. 2021. "Incorporation of Thermal Hydraulic Models for Thermal Power Dispatch into a PWR Power Plant Simulator." Report No. INL/EXT-21-63226, Idaho Falls, Idaho: Idaho National Laboratory.
- [12] De Kock, Jan and Cobus Strauss. 2004. *Practical Power Distribution for Industry*. Amsterdam; Boston: Newnes/Elsevier, 2004.
- [13] Arif, S. and T. Aziz, 2017. "Study of Transient Stability with Battery Energy Storage Systems in Renewable Integrated Islanded Microgrid." 2017 IEEE International WIE Conference on Electrical and Computer Engineering (WIECON-ECE), Dehradun, India, 2017, pp. 266-269, doi: 10.1109/WIECON-ECE.2017.8468905
- [14] Casteel, M., Hydrogen and Thermal Systems Engineer, Idaho National Laboratory, email message to author, April 2024.

- [15] European Commission. 2023. "HIAD 2.1, The Hydrogen Incident and Accidents Database." European Commission, Joint Research Centre. Published May 12, 2023. Accessed February 11, 2024. https://minerva.jrc.ec.europa.eu/en/shorturl/capri/hiadpt
- [16] Wen, Jennifer X., Marta Marono, Pietro Moretto, Ernst-Arndt Reinecke, Pratap Sathiah, Etienne Studer, Elena Vyazmina, and Daniele Melideo. 2022. "Statistics, Lessons Learned and Recommendations from Analysis of HIAD 2.0 Database." International Journal of Hydrogen Energy 47 (38): 17082-17096. https://doi.org/10.1016/j.ijhydene.2022.03.170
- [17] National Fire Protection Association. 2023. "NFPA 2: Hydrogen Technologies Code." National Fire Protection Association." Current edition: 2023. Accessed February 12, 2024. https://www.nfpa.org/codes-and-standards/2/nfpa-2
- [18] U.S. Department of Labor. n.d. "Fatality and Catastrophe Investigation Summaries." U.S. Department of Labor Occupational Safety and Health Administration. Retrieved November 11, 2023. https://www.osha.gov/ords/imis/accidentsearch.html
- [19] ARIA. n.d. "ARIA: feedback on technological accidents." Bureau d'Analyse des Risques et Pollutions Industriels. https://www.aria.developpement-durable.gouv.fr/?lang=en. Retrieved February 9, 2024.
- [20] U.S. Chemical Safety Board. n.d. "Investigations." U.S. Chemical Safety and Hazard Investigation Board. https://www.csb.gov/investigations/. Retrieved February 9, 2024.
- [21] U.S. Chemical Safety Board. 2010. "Seven Key Lessons to Prevent Worker Deaths During Hot Work in and Around Tanks: Effective Hazard Assessment and Use of Combustible Gas Monitoring Will Save Lives." Chemical Safety and Hazard Investigation Board, Report No. 2009-01-SB, February 2010. https://www.csb.gov/userfiles/file/csb%20hot%20work%20safety%20bulletin%20embargoed%20until%2010%20a_m_%203_4_10%281%29.pdf
- [22] U.S. Chemical Safety Board. 2007. "Investigation Report-Refinery Explosion and Fire in BP Texas City." U.S. Chemical Safety and Hazard Investigation Board. Published March 23, 2005. Accessed June 12, 2024. https://www.csb.gov/bp-america-texas-city-refinery-explosion/.
- [23] Cheng, W.-C., et al. 2020. "Review and categorization of existing studies on the estimation of probabilistic failure metrics for Reactor Coolant Pressure Boundary piping and steam generator tubes in Nuclear Power Plants." Progress in Nuclear Energy 118 (2020): 103105.
- [24] Novotny, V., J. Kim, S. Creasman, L. Williams, and D. Mikkelson, 2024. "Preliminary Process and Instrumentation Design of Advanced Reactor Integration with Refineries and Hydrogen Production Facilities." Report no. INL/RPT-24-76271, Idaho Falls, Idaho: Idaho National Laboratory. https://doi.org/10.2172/2305387
- [25] Occupational Safety and Health Administration. 1995. "8-hour total weight average (TWA) permissible exposure limit (PEL)." U.S. Department of Labor. Published October 6, 1995. Accessed June 12, 2024. https://www.osha.gov/laws-regs/standardinterpretations/1995-10-06-3#:~:text=%22TWA%20is%20the%20employee's%20average,risk%20of%20adverse%20health%20effects
- [26] Occupational Safety and Health Administration. n.d. "Chemical Hazards and Toxic Substances: What other common terms are used when discussing chemical hazards or toxic substances? Ceiling Limit." U.S. Department of Labor. Accessed June 12, 2024. https://www.osha.gov/chemical-hazards#:~:text=Ceiling%20Limit,see%20the%20OSHA%20Technical%20Manual

- [27] United Nations. 2019. *Globally Harmonized System of Classification and Labelling of Chemicals* (*GHS*). Report No. ST/SG/AC.10/30, Rev.8, United Nations, New York and Geneva. https://unece.org/fileadmin/DAM/trans/danger/publi/ghs/ghs_rev08/ST-SG-AC10-30-Rev8e.pdf
- [28] The Engineering ToolBox. 2018. *Methanol Thermophysical Properties*. Published 2018. Accessed July 5, 2024. https://www.engineeringtoolbox.com/methanol-methyl-alcohol-properties-CH3OH-d 2031.html
- [29] NIH National Library of Medicine, National Center for Biotechnology Information. 2024. "PubChem Compound Summary for CID 887, Methanol." PubChem. Modified September 7, 2024. Accessed August 22, 2024. https://pubchem.ncbi.nlm.nih.gov/compound/Methanol
- [30] Idaho National Engineering and Environmental Laboratory. 1994. "Improved Estimates of Separation Distances to Prevent Unacceptable Damage to Nuclear Power Plant Structures from Hydrogen Detonation for Gaseous Hydrogen Storage." Report No. EGG-SSRE-9747, Idaho Falls, Idaho: Idaho National Engineering and Environmental Laboratory. https://doi.org/10.2172/10167006
- [31] Smith, C., S. Beck, and B. Galyean. 2005. "Separation Requirements for a Hydrogen Production Plant and High-Temperature Nuclear Reactor." Report No. INL/EXT-05-00137, Idaho Falls, Idaho: National Laboratory. https://doi.org/10.2172/911611
- [32] Vedros, K. G. and C. Otani. 2020. "Preliminary Probabilistic Risk Assessment of a Light Water Reactor Supplying Process Heat to a Hydrogen Production Plant." Report No. INL/EXT-19-55884, Revision 1, Idaho Falls, Idaho: Idaho National Laboratory.
- [33] IES Utilities Inc. 1995. "Duane Arnold Energy Center Individual Plant Examination of External Events," NEI 05-01, Nuclear Energy Institute.
- [34] Budnitz, R. J., Amico, P. J., Corneli, C. A., Hall, W.J., Kennedy, R.P., Reed, J. W., and Shinozuka, M. 1985. "An Approach to the Quantification of Seismic Margins in Nuclear Power Plants." Report No. NUREG/CR-4334, Lawrence Livermore National Laboratory.
- [35] Casal, J. 2018. Evaluation of the Effects and Consequences of Major Accidents in Industrial Plants, 2nd Edition. Elsevier, 2018. https://doi.org/10.1016/C2016-0-00740-4
- [36] Ahumada, Cassio Brunoro, Frank-Ioannis Papadakis-Wood, Pratik Krishnan, Shuai Yuan, Noor Quddus, M. Sam Mannan, and Qingsheng Wang. 2020. "Comparison of Explosion Models for Detonation Onset Estimation in Large-scale Unconfined Vapor Clouds." *Journal of Loss Prevention in the Process Industries*, vol. 66, p. 104165. https://doi.org/10.1016/j.jlp.2020.104165
- [37] Ehrhart, B. D., and Hecht, E. S. 2022. "Hydrogen Plus Other Alternative Fuels Risk Assessment Models (HyRAM+) Version 5.0 Technical Reference Manual." Report No. SAND2022-16425, Albuquerque, New Mexico: Sandia National Laboratories. https://doi.org/10.2172/1900089
- [38] U.S. CFR. 2004 "40 CFR Part 68: Chemical Accident Prevention Provisions. Sub-chapter C" U.S. Code of Federal Regulations, April 9, 2004. https://www.ecfr.gov/current/title-40/chapter-I/subchapter-C/part-68
- [39] Center for Chemical Process Safety. 2010. *Guidelines for Vapor Cloud Explosion, Pressure Vessel Burst, BLEVE, and FLASH Fire Hazards*, Second Edition, John Wiley & Sons, New York, 2010. https://onlinelibrary.wiley.com/doi/book/10.1002/9780470640449
- [40] Factory Mutual Global, Property Loss Prevention Data Sheets, 7–42. 2008. "Guidelines for Evaluating the Effect of Vapor Cloud Explosions Using a TNT Equivalency Method." Factory Mutual Insurance Company, Johnston, RI, May 2008. https://www.nrc.gov/docs/ML1422/ML14224A361.pdf

- [41] Jones, R., W. Lehr, D. S. Beatty, and R. M. Reynolds. 2013. "ALOHA 5.4.4 Technical Documentation." U.S. National Oceanic and Atmospheric Administration, Seattle, WA, 2013. https://response.restoration.noaa.gov/sites/default/files/ALOHA_Tech_Doc.pdf
- [42] Louie, M. S., D. Brooks, and A. M. Glover. 2024. "Topical Analyses Related to Co-located Industrial Facilities at Nuclear Power Plants." Report No. SAND2024-08110, Albuquerque, New Mexico: Sandia National Laboratories. https://doi.org/10.2172/2429909
- [43] Camfil. n.d. "Stinger Explosion Isolation Valves." Camfil Air Pollution Control. Accessed September 6, 2024. https://camfilapc.com/products/accessories/stinger.
- [44] CV Technology. n.d. "Active Chemical Suppression." CV Technology. Accessed September 6, 2024. https://cvtechnology.com/explosion/chemical-suppression
- [45] Block Moulds. n.d. "Blast Proof Walls at Affordable Prices." Block Moulds. Accessed September 6, 2024. https://www.blockmoulds.com/applications/blast-walls
- [46] Bowsers. n.d. "Blast Walls." Bowsers. Accessed September 6, 2024. https://bowsers.com.au/capabilities/blast-walls
- [47] Goel, M. D. and V. A. Matsagar. 2014. "Blast-Resistant Design of Structures." Practice Periodical on Structural Design and Construction, vol. 19, no. 2, p. 04014007, May 2014, doi: https://doi.org/10.1061/(asce)sc.1943-5576.0000188
- [48] U.S. Environmental Protection Agency. 2023. "ALOHA software." U.S. Environmental Protection Agency. Accessed February 7, 2024. https://www.epa.gov/cameo/aloha-software
- [49] Christian, R. and H.G. Kang. 2015. "Radiological Impact Assessment of a Prolonged Release from a Pyroprocessing Facility." Transactions of the Korean Nuclear Society Spring Meeting, Jeju, Republic of Korea, May 2015. https://www.kns.org/files/pre_paper/33/15S-605RobbyChristian.pdf
- [50] Shi, Y., C. Xie, Z. Li, and Y. Ding. 2021. "A quantitative correlation of evaluating the flame speed for the BST method in vapor cloud explosions." Journal of Loss Prevention in the Process Industries, Vol. 73, November 2021. https://doi.org/10.1016/j.jlp.2021.104622
- [51] Christian, R., C.M. Otani, K. Vedros, and A.M. Glover. 2024. "Preliminary Risk Analysis of Nuclear Co-located Hydrogen and Syngas Production." Proceedings of Pacific Basin Nuclear Conference, Idaho Falls, October 2024 (forthcoming).
- [52] MRCC. n.d. "cli-MATE: MRCC Application Tools Environment." Midwestern Regional Climate Center, Accessed July 18, 2024. https://mrcc.purdue.edu/CLIMATE/
- [53] U.S. NOAA. n.d. "Levels of Concern Guide." U.S. National Oceanic and Atmospheric Administration. Accessed February 7, 2024. https://cameochemicals.noaa.gov/help/reference/locs_guide/overview.htm
- [54] Blankenhagel, P., K.-D. Wehrstedt, K. B. Mishra, and J. Steinbach. 2018. "Thermal Radiation Assessment of Fireballs Using Infrared Camera." Journal of Loss Prevention in the Process Industries, vol. 54, pp. 246–253, Jul. 2018, doi: https://doi.org/10.1016/j.jlp.2018.04.008
- [55] Edwards, T. 2017. "Reference Jet Fuels for Combustion Testing." American Institute of Aeronautics and Astronautics. Accessed July 15, 2024. https://www.caafi.org/news/pdf/Edwards_AIAA-2017-0146_Reference_Jet_Fuels.pdf
- [56] U.S. NOAA. 1999. "Naphtha: VM & P." U.S. National Oceanic and Atmospheric Administration. Published June 1999. Accessed July 8, 2024. https://cameochemicals.noaa.gov/chris/NVM.pdf

- [57] Andrade, R. D. A., et al. 2011. "Heat of combustion of biofuels mixed with fossil diesel oil." Journal of Thermal Analysis and Calorimetry, vol. 106, no. 2, pp. 469–474. https://doi.org/10.1007/s10973-011-1408-x
- [58] U.S. NOAA. 1999. "Methyl Alcohol." U.S. National Oceanic and Atmospheric Administration. Published June 1999. Accessed July 15, 2024. https://cameochemicals.noaa.gov/chris/MAL.pdf
- [59] LaChance, Jeffrey, Andrei Tchouveley, and Angunn Engebo. 2011. "Development of Uniform Harm Criteria for Use in Quantitative Risk Analysis of the Hydrogen Infrastructure." International Journal of Hydrogen Energy 36 (3): 2381–2388. https://doi.org/10.1016/j.ijhydene.2010.03.139
- [60] Vanzi, I. 1996. "Seismic Reliability of Electric Power Networks: Methodology and Application." Structural Safety 18: 311–327. https://doi.org/10.1016/S0167-4730(96)00024-0
- [61] Chalishazar, V. H., et al. 2020. "Connecting Risk and Resilience for a Power System Using the Portland Hills Fault Case Study." Processes 8, no.10: 1200. https://doi.org/10.3390/pr8101200
- [62] Jiao, A., D. Ricks, T. Remick and B. J. Hansen, "Establishment of Control Room Habitability Analysis Methodology Postulating the Event of Pressurized Tank/System of Hazardous Gas Release." Nuclear Science and Engineering, 197:11, 2830–2839, DOI: 10.1080/00295639.2023.2171274, 2023
- [63] Ma, Z., T. E. Wierman, and K. J. Kvarfordt. 2021. "Industry-Average Performance for Components and Initiating Events at U.S. Commercial Nuclear Power Plants: 2020 Update." INL/EXT-21-65055, Idaho National Laboratory. https://doi.org/10.2172/1847110
- [64] IEEE. 2007. "IEEE Recommended Practice for the Design of Reliable Industrial and Commercial Power Systems." IEE Std 493-2007, Idaho National Laboratory. https://doi.org/10.1109/IEEESTD.2007.380668
- [65] NRC. 2021. "Application for Amendment of License or Construction Permit at Request of Holder," 10 CFR 50.90, U.S. Nuclear Regulatory Commission. Accessed August 8, 2024. https://www.nrc.gov/reading-rm/doc-collections/cfr/part050/part050-0090.html
- [66] Swindlehurst, G. 2019. "Safety Evaluation of Modification for Process Steam Supply Capability in Pressurized Water Reactors, Rev. 1." GS Nuclear Consulting, LLC, Idaho National Laboratory.
- [67] Sargent & Lundy. 2023c. "Nuclear Power Plant Pre-Conceptual Licensing Support for Large-Scale 500-MWnom Hydrogen Production Facility." SL-017513, Revision 1.
- [68] Nuclear Energy Institute (NEI). 2000. "Guidelines for 10 CFR 50.59 Implementation." 96-07, Revision 1. November 17, 2000. ADAMS Accession No. ML003771157.
- [69] NRC. 2000. "Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments." Regulatory Guide (RG) 1.187, November 2000 (ADAMS Accession No. ML003759710).
- [70] Remer, J., J. Cadogan, K. Vedros. 2024. "Nuclear Integrated Hydrogen Code and Licensing Separation Distance Considerations." INL/RPT-24-04914, Idaho National Laboratory.
- [71] U.S. CFR. n.d. "National Fire Protection Association Standard 805." U.S. Code of Federal Regulations 10 CFR 50.48(c). https://www.nrc.gov/reading-rm/doc-collections/cfr/part050/part050-0048.html
- [72] NRC. 1983. "Safety Goals for Nuclear Power Plant Operation." NUREG-0880, Revision 1, Accessed August 26, 2024. https://www.nrc.gov/docs/ML0717/ML071770230.pdf
- [73] Nuclear Regulatory Commission. 2017a. "10 CFR 50.91 Notice for Public Comment; State Consultation." U.S. Code of Federal Regulations, https://www.nrc.gov/reading-rm/doc-collections/cfr/part050/part050-0091.html

[74] Nuclear Regulatory Commission. 2017b. "10 CFR 50.92 Issuance of Amendment," U.S. Nuclear Regulatory Commission. Accessed October 8, 2020. https://www.nrc.gov/reading-rm/doc-collections/cfr/part050/part050-0092.html

Page left intentionally blank.

Appendix A Generic PWR PRA Model

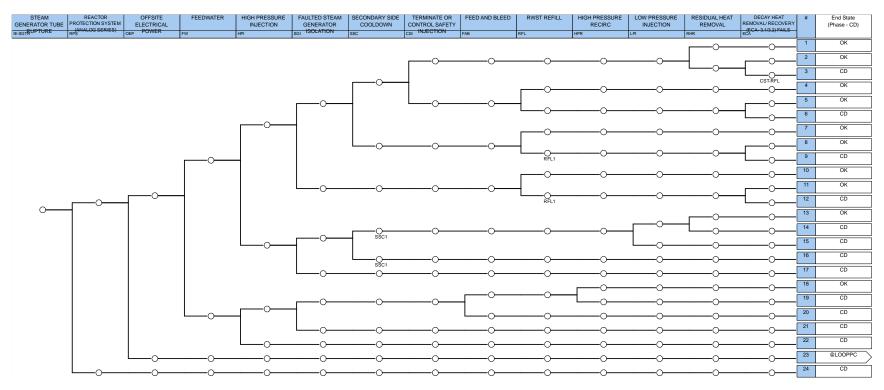


Figure A-1. SGTR ET.

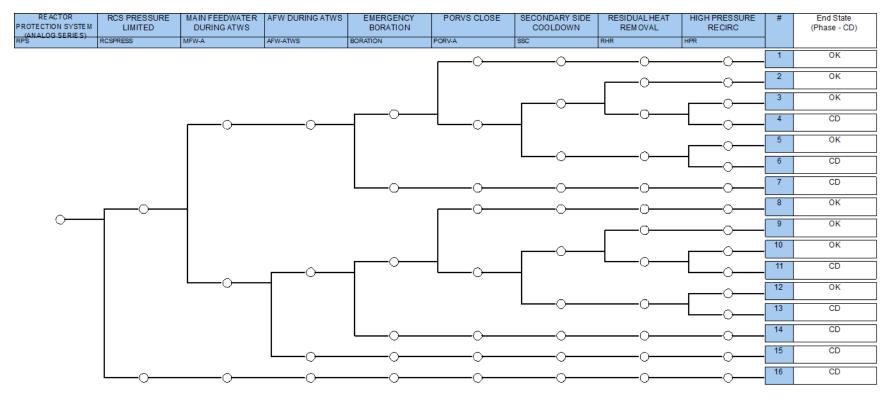


Figure A-2. ATWS ET.

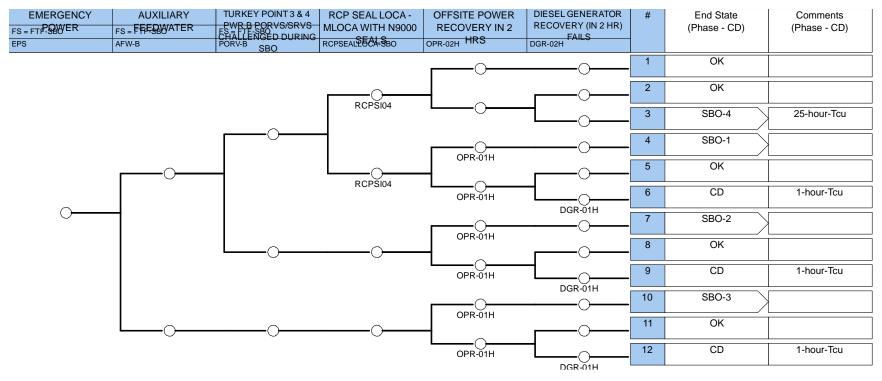


Figure A-3. Station blackout (SBO) ET.

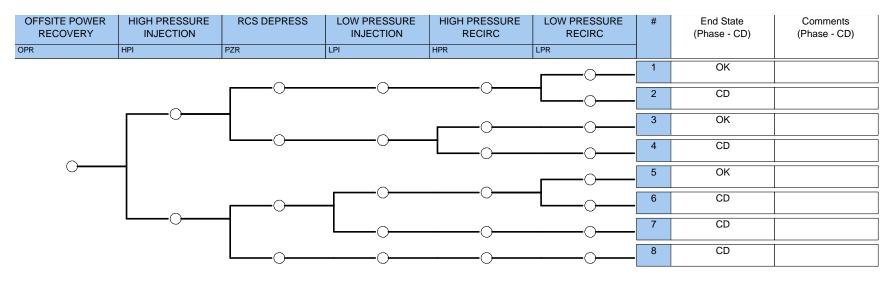


Figure A-4. SBO-1 ET.

OFFSITE POWER RECOVERY	HIGH PRESSURE INJECTION	HIGH PRESSURE RECIRC	#	End State (Phase - CD)	Comments (Phase - CD)
OPR	HPI	HPR			
			1	OK	
\circ	-		2	CD	
		<u> </u>	3	CD	

Figure A-5. SBO-2 ET.

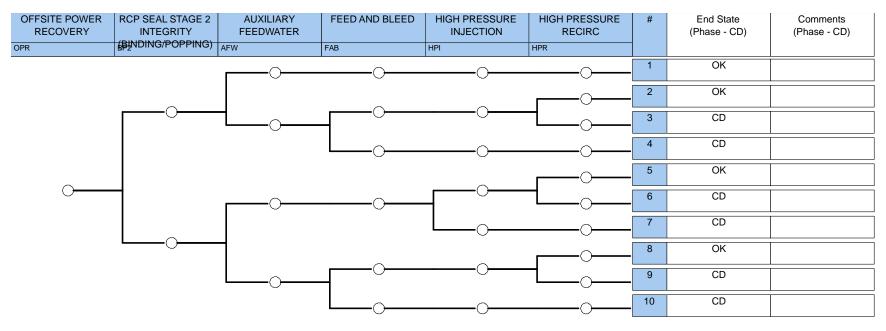


Figure A-6. SBO-3 ET.

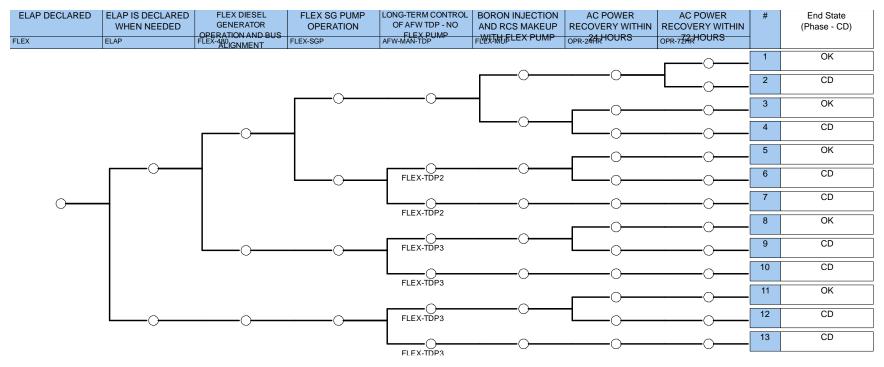


Figure A-7. SBO-4 ET.

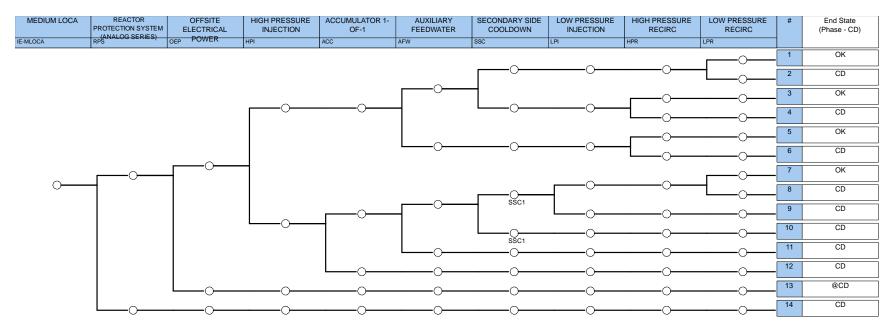


Figure A-8. Medium loss-of-coolant accident ET.

Page intentionally left blank

Appendix B Generic BWR PRA Model

This appendix shows BWR ETs, which are transfers of the accident mitigation ETs described in the body of this report. The general plant transient ET previously shown in Section 6.3 is truncated and displayed in several parts here for better readability. The one stuck-open relief valve ET is shown in multiple parts for the same reason.

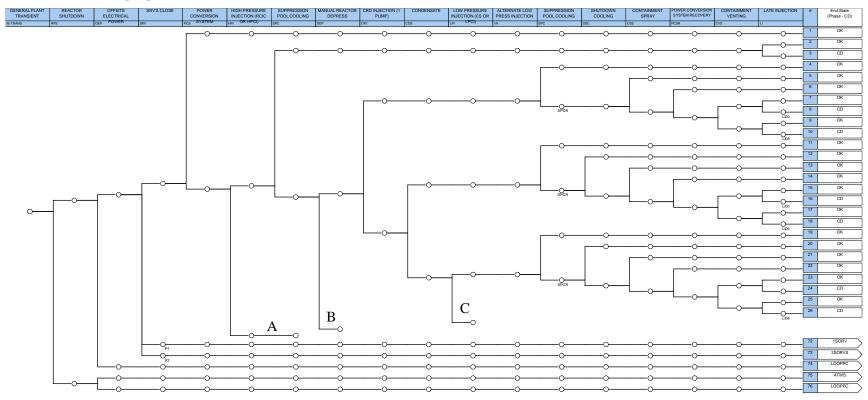


Figure B-1. General plant transient ET (IE-TRANS) Part 1, showing three truncated branches (i.e., branch A, B, and C).

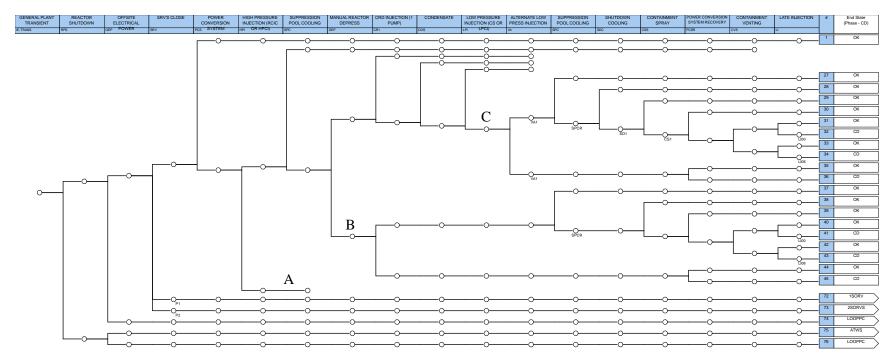


Figure B-2. General plant transient ET (IE-TRANS) Part 2, revealing branch B and C.

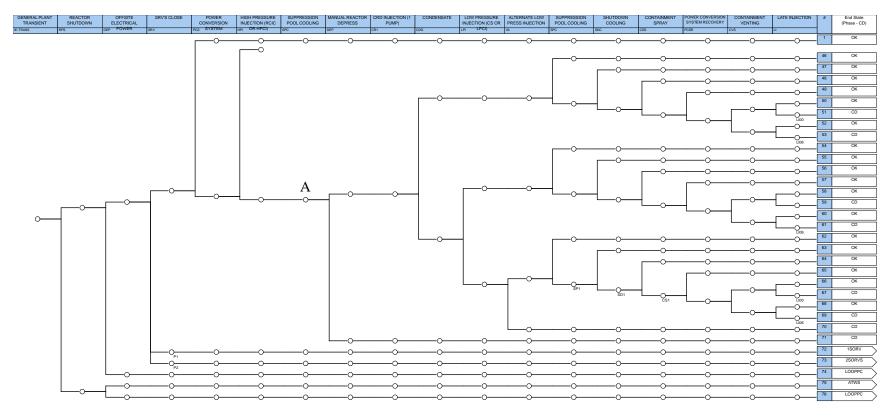


Figure B-3. General plant transient ET (IE-TRANS) Part 3, revealing branch A.

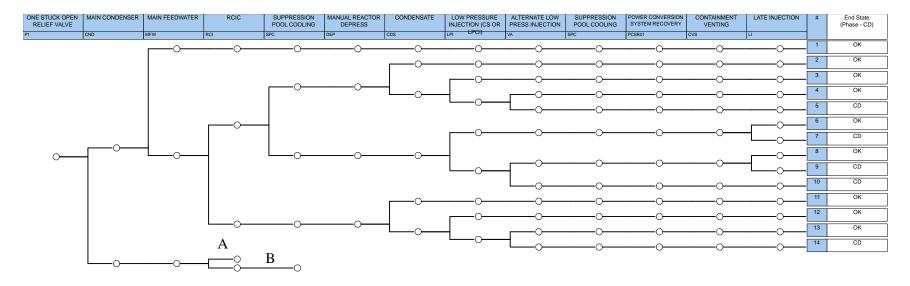


Figure B-4. One stuck-open relief valve ET (P1) Part 1, showing a truncated branch.

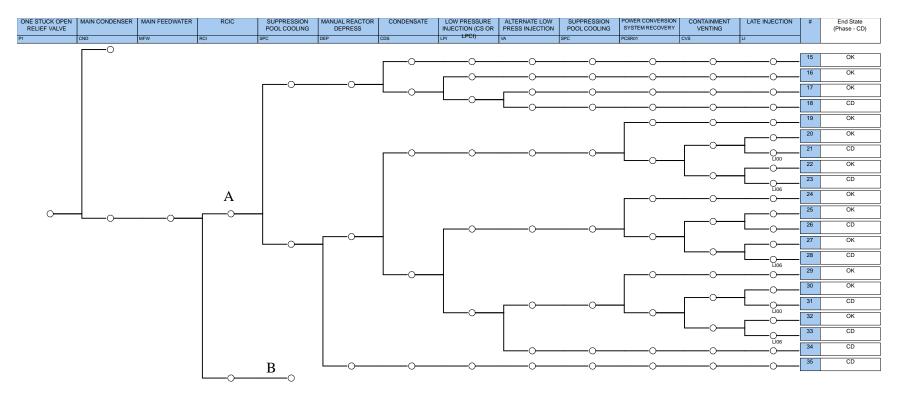


Figure B-5. One stuck-open relief valve ET (P1) Part 2, revealing branch A.

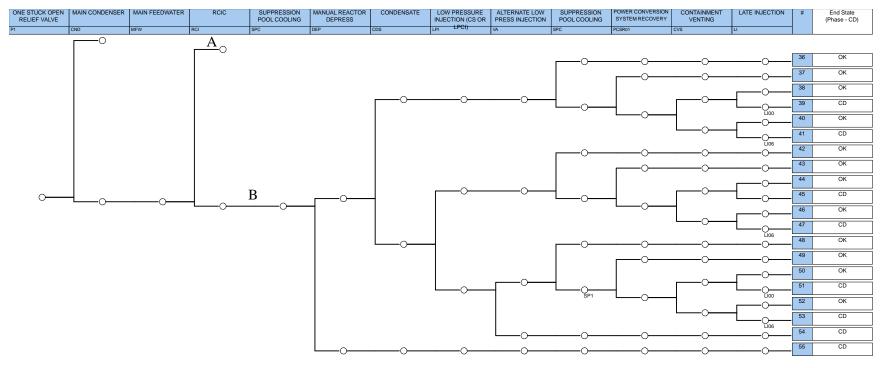


Figure B-6. One stuck-open relief valve ET (P1) Part 3, revealing branch B.

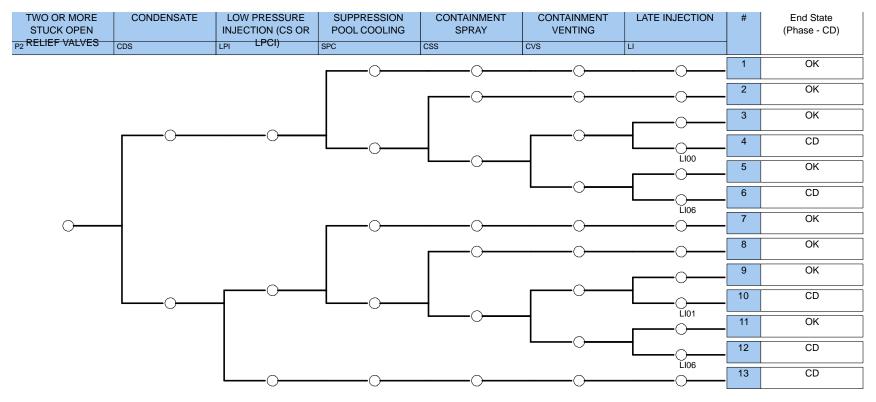


Figure B-7. Two or more stuck-open relief valves (P2).

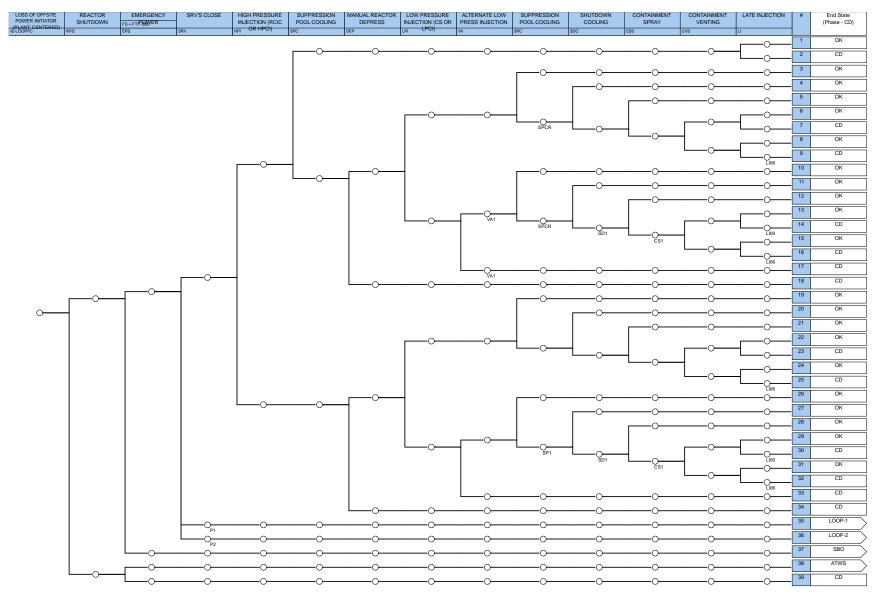


Figure B-8. LOOP (plant-centered) ET (IE-LOOPPC).

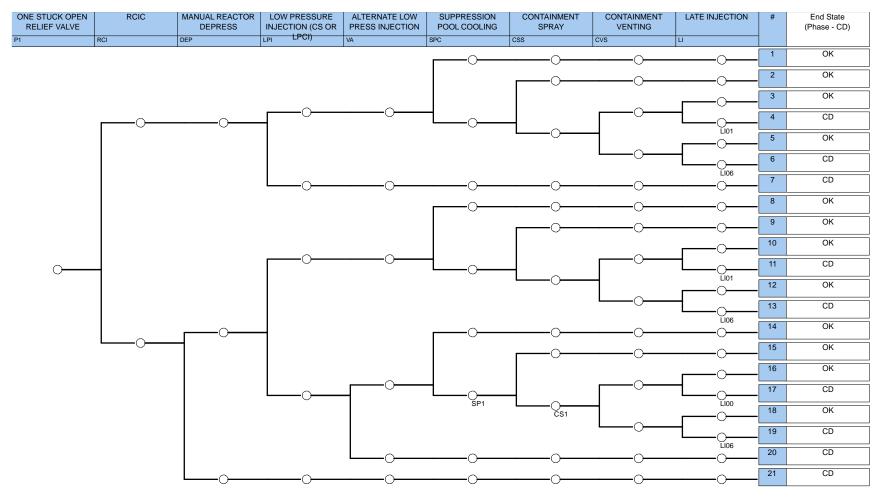


Figure B-9. LOOP-1 ET (P1).

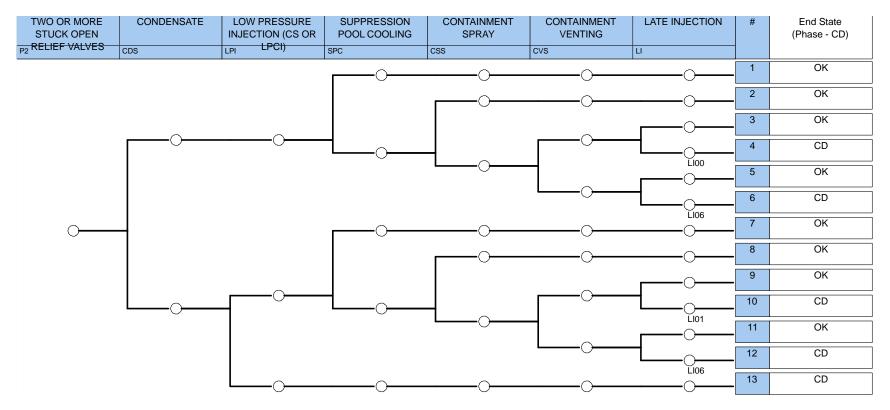


Figure B-10. LOOP-2 ET (P2).

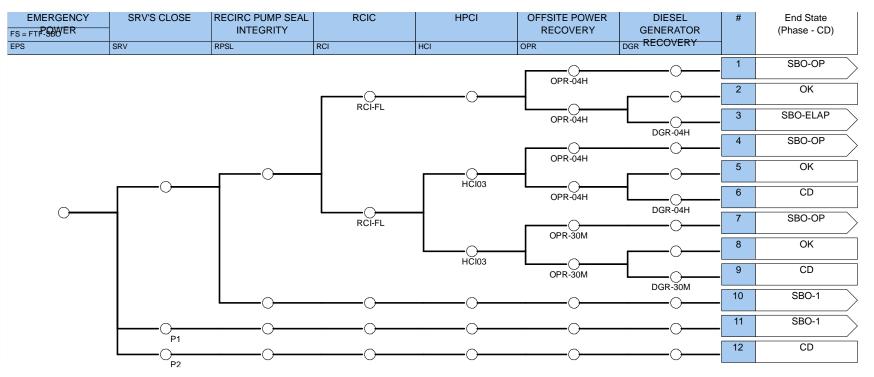


Figure B-11. SBO ET.

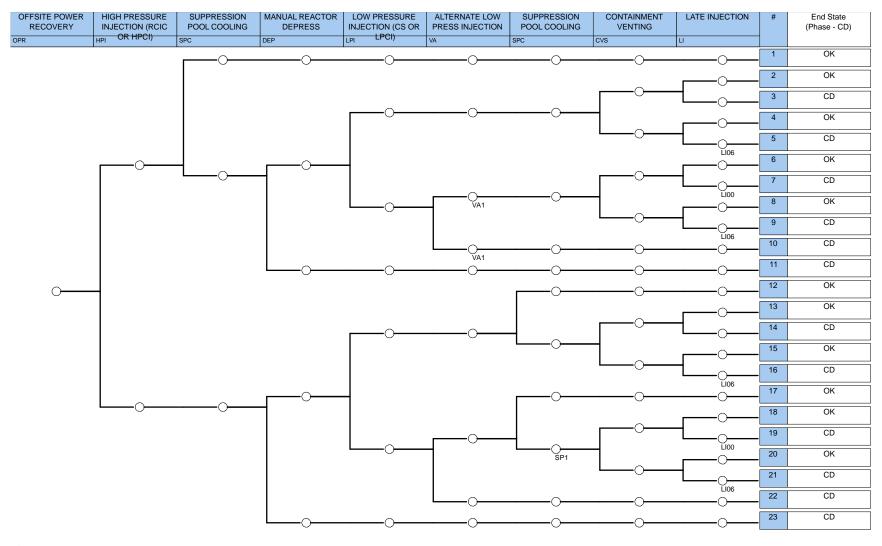


Figure B-12. SBO-OP ET.

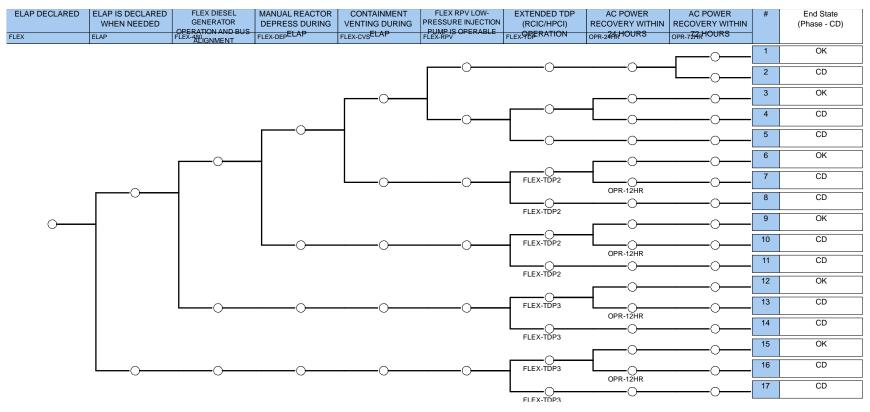


Figure B-13. SBO-ELAP ET.

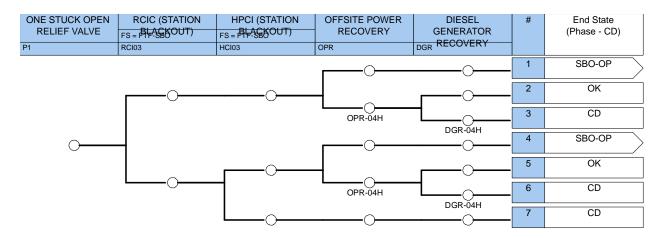


Figure B-14. SBO-1 ET.

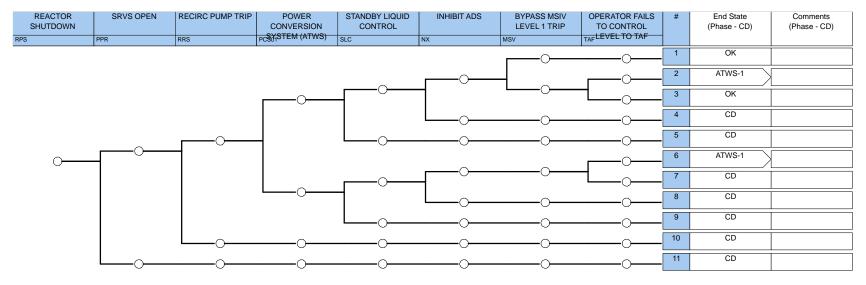


Figure B-15. ATWS ET.

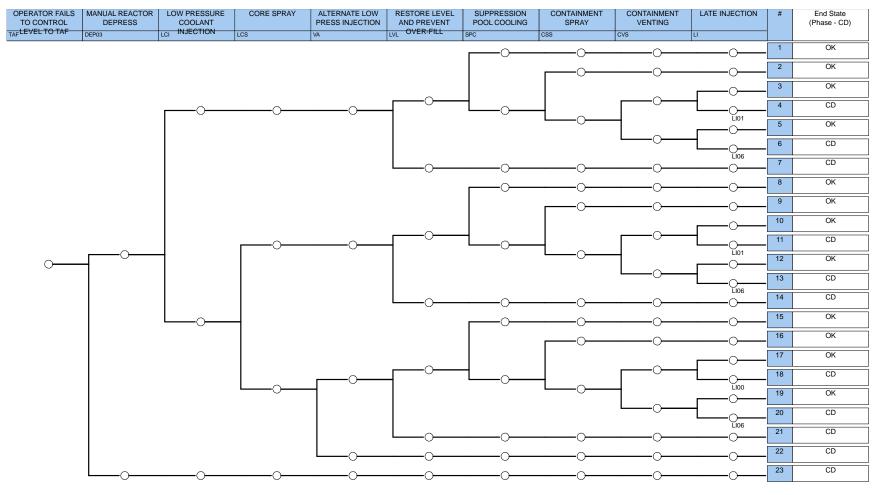


Figure B-16. ATWS-1 ET.

Page intentionally left blank

Appendix C FMEA Criteria

The FMEA results for BWR or PWR co-located with industrial facilities (refinery, methanol, and wood pulp and paper mill) and one specific process that is found in methanol and refinery use (syngas) are presented on the following appendices. All appendices are organized by the impacted subject of interest (e.g., nuclear power plant, syngas facility itself, public safety and perception, and economic impact). It is recognized that economic impact on either the NPP or the industrial facility will affect both the industrial facility and the NPP.

The scoring criteria used for all FMEAs followed Tables C-1 and C-2.

Table C-1. Scoring criteria for FMEA ranking categories.

Score	Severity	Frequency	Detection
1	Little to no impact	No incidences recorded or able to avoid by siting at safe siting distance	Always quickly detected (sensor available in correct spot)
2	Small impact	1 incident recorded	Detected with aging sensor
3	Indirect impact (e.g., lower security)	1E-5 per facility year	
4	Unexpected but unhindered shutdown	1E-4 per facility year	
5	Potentially hindered shutdown and equipment damage	1E-3 per facility year	Detection available in other part of system (e.g., condensate for NPP)
6	Hindered shutdown and operations	1E-2 per facility year	
7	Damage debris, damage, personnel injuries	1E-1 per facility year	
8	Personnel fatalities and hindered shutdown	1 or more per year	
9	Severely hindered shutdown	3 or more per year	
10	Maximum impact, station blackout conditions	5 or more per year	Never detected and no sensor available

Table C-2. Risk Priority Number (RPN) acronym descriptions

Acronym	Range	Description
S	1-10	Severity (1 = most severe)
F	1-10	Frequency
D	1-10	Detection $(1 = \text{easiest to detect})$

Page intentionally left blank.

Appendix D FMEA Results for SynGas Production

The FMEA results for syngas production are listed in the following tables.

Table D-1. Nuclear power plant based FMEA results for SynGas.

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
External Power	Loss of offsite power	H ₂ detonation at HTEF Syngas deflagration near both NPP input feed	S = 9 F = 1 D = 1 Total = 9 S = 5 F = 3	Severity highly dependent on NPP. Number of plants where a LOOP is a really bad day. It depends on the configuration of emergency power. The FMEA team listed severity as a range between 3 to 9. The highest number listed is used here. Must also look at next-most fragile components beyond the transmission towers and auxiliary transformers to see if they are sited at critical distances. Concentric rings of overpressure can help visualize. Syngas facility does not require co-location because it does not need steam from the nuclear power plant. However,
		transmission towers or cables	D = 1 Total = 15	syngas is a denser than air gas. If it leaks, it can be blown by the wind, probably toward nearby power transmission lines, until it meets an ignition source. Therefore, the hazard is not localized to the leakage point. On the other hand, syngas is unlikely to experience DDT resulting in a significant overpressure, although a subsonic fire may still damage power cables and equipment causing a power loss. With such considerations, the severity rank is less than hydrogen's, while the frequency rank is higher.
Primary loop transport of process steam	Loss of thermal output to HTEF Damage to turbine building equipment, possibly safety power buses, depending on the plant	Pipe Rupture after MSIV Operational vibration seismic, and erosion	S = 4 F = 2 D = 1 Total = 8	If safety buses are in the turbine bldg, then site the HES outside of turbine bldg. Another advantage to having the reboilers in their own building is lower temperatures in turbine building.

Table D-1 Nuclear power plant based FMEA results for SynGas Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
Spent fuel storage (dry)	Cask tip-over due to overpressure or cask structural degradation due to heat flux	H ₂ detonation at HTEF	S = 7 F = 1 D = 1 Total = 7	Possible damage to storage building, if used. H ₂ Facility must have sufficient separation such that dry casks cannot be damaged.
		Syngas deflagration at spent fuel storage area	S = 1 F = 1 D = 1 Total = 1	Syngas has a relatively low heat of combustion compared to other fuels, including wood and coal. So, the heat generated from syngas fire is unlikely to cause significant damage to spent fuel casks. Therefore, the severity ranking is 1.
Electrical load to HTEF	Prompt loss of behind-the-meter electrical load to HTEF causes disruptive feedback to turbine	Unexpected immediate HTEF shutdown	S = 7 F = 1 D = 1 Total = 7	Would require failure of switchyard protection. The frequency is very low.
Makeup water pipeline	Loss of makeup water supply to spray ponds/cooling towers due to damaged pipeline	H ₂ detonation at HTEF	S = 5 $F = 1$ $D = 1$ $Total = 5$	Possible seismic upset to pipeline to ultimate heat sink.
H ₂ in NPP process	Increased levels of H ₂ in steam return	H ₂ piped back to NPP	S = 1 $F = 1$ $D = 5$ $Total = 5$	H ₂ levels are low and are already in risk assessments of applicable NPPs.
Spray pond	Degradation of ultimate heat sink	H ₂ detonation at HTEF	S = 3 F = 1 D = 1 Total = 3	Debris and above-water spray mechanisms, ultimate heat sink With adequate protection through distance and/or barriers this would be a severity of zero.
Cooling tower pond	Degradation of ultimate heat sink	H ₂ detonation at HTEF	S = 3 F = 1 D = 1 Total = 3	Debris in ultimate heat sink With adequate protection through distance and/or barriers this would be a severity of zero.
Non-Safety Service water pump house	Damage and/or loss of service water building and equipment	H ₂ detonation at HTEF	S = 2 F = 1 D = 1 Total = 2	As sited at calculated safe distance HTEF to pump house or with blast barrier.

Table D-1 Nuclear power plant based FMEA results for SynGas Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
		Syngas fire within NPP complex	S = 4 F = 1 D = 1 Total = 4	While H ₂ detonation hazard is controlled through a safe separation distance between nuclear power plant and possible leakage points in the HTEF, syngas may travel downwind of its leakage point until it meets an ignition source in the NPP complex. Therefore, the severity of such an event is predicted to be higher than H ₂ detonation.
Forced air cooling for non- safety buildings	Damage and/or loss of NPP building HVAC equipment. Reactor building, admin building, etc	H ₂ detonation at HTEF	S = 2 F = 1 D = 1 Total = 2	Can affect human operations. May have to shut down reactor.
		Syngas fire in NPP complex	S = 2 F = 1 D = 1 Total = 2	
NPP & H ₂ administrative support	Damage to staffs' cars, office buildings and equipment	H ₂ detonation at HTEF	S = 2 F = 1 D = 1 Total = 2	While not directly related to NPP safety, damage to support buildings can affect operations.
		Syngas fire in NPP complex	S = 2 F = 1 D = 1 Total = 2	
Physical protection	Damage to intrusion sensors, or triggering multiple false alarms, or empty guard posts due to evacuation	H ₂ detonation at HTEF	S = 1 $F = 1$ $D = 1$ $Total = 1$	Lowered physical protection profile can lead to an opening for terrorist activity.
		Syngas dispersion reaching NPP security perimeters	S = 2 F = 1 D = 1 Total = 2	Syngas is toxic. Therefore, syngas dispersion in a nuclear power plant will lead to a longer evacuation of outdoor staffs compared to an instantaneous hydrogen blast explosion. Therefore, the severity ranking is 2.
NPP operation	Limited outdoor operation due to toxic concentration of syngas	Syngas dispersion reaching NPP complex	S = 3 $F = 1$ $D = 1$ $Total = 3$	Similar to the above. Syngas' toxicity can prevent outdoor operations such as maintenance actions.

Table D-1 Nuclear power plant based FMEA results for SynGas Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
Steam diversion load roughly 5% thermal	Loss of 5% load immediately	Pipe Rupture after MSIV	S = 0 $F = 1$	NPP can handle up to 30% prompt load loss, so not a hazard.
		Operational vibration seismic, and erosion	D = 1 $Total = 0$	
External Supply Tanks integrity	Damage to CST, other supply tanks	H ₂ detonation at HTEF	S = 0 F = 1 D = 1 Total = 0	As sited at calculated safe distance NPP to HTEF.
		Syngas fire in NPP complex	S = 7 F = 1 D = 1 Total = 7	Syngas facility does not require co-location because it does not need steam from the nuclear power plant. However, syngas is a denser than air gas. If it leaks, it can be blown by the wind toward NPP complex. Assuming there is a significant distance between the facilities, it is unlikely for the wind to blow in the right direction toward NPP, and for syngas concentration to still be above the LEL at the NPP complex. Therefore, the frequency ranking is assigned as 1.
Critical structure integrity	Damage to reactor building walls	H ₂ detonation at HTEF	S = 0 F = 1 D = 1 Total = 0	As sited at calculated safe distance NPP to HTEF.

Table D-2. Industrial customer based FMEA results for syngas

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Industrial Plant	General Notes
Hydrogen Transport by Truck	H ₂ detonation at HTEF	Fueling accident, fitting leak, valve leak, etc., along with hydrogen capture and ignition source	S = 10 F = 2 D = 1 Total = 20	Most severe hydrogen-based industrial accidents happen during fueling operations. Preventing accumulation opportunities through design is a key mitigator.
H ₂ Storage at plant	H ₂ detonation at HTEF	Tank leak/rupture with ignition source Forklift or other industrial equipment tears a hole in the tank. Possible high-wind missile strike.	S = 10 F = 2 D = 1 Total = 20	Severity based on volume and pressure of tank and distance. Very hard to determine frequency of a rupture event from industrial accident. Consequences are identified, but there is not a historical instance of a rupture with a detonation, only a deflagration.
H ₂ production	Electrolysis stacks damaged/toppled if stacked	High winds or tornado	S = 10 F = 2 D = 1 Total = 20	Frequency is dependent upon location. Proper design can overcome the hazard.
H ₂ Storage at plant	Tank rupture with ignition source H ₂ fire at HTEF	Forklift or other industrial equipment tears a hole in the tank. Possible high-wind missile strike.	S = 10 F = 1 D = 1 Total = 10	Severity based on volume and pressure of tank. Potential heat flux should be a consideration in design and placement of barriers.
Multiple	H ₂ detonation at HTEF	Piping or tank leak/rupture along with an ignition source	S = 10 F = 1 D = 1 Total = 10	Pipe rupture may cause a pipe whip and impact nearby equipment and personnel. Any flow through crack is expected to be small and may disperse in atmosphere.
Thermal delivery to hydrogen plant	Heat Exchanger Leak, steam leak, kinetic and thermal hazard	Overpressurization of HTEF supply loop - failure of relief valve	S = 5 F = 2 D = 1 Total = 10	Relief valve in the HTEF loop within the HTEF.
H ₂ Production	Electrolysis stacks damaged/toppled if stacked	H ₂ detonation at HTEF	S = 10 F = 1 D = 1 Total = 10	Severity based on severity and location (within stack, in system pipelines, in heat exchangers, etc.) of detonation, either way, production of H_2 would be halted. Design of facility stacking to wind/seismic codes minimizes this hazard.

Table D-2. Industrial customer based FMEA results for syngas Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Industrial Plant	General Notes
Multiple	H ₂ fire at HTEF Heat flux damage to nearby personnel, equipment, and structures	Piping or tank leak/rupture along with an ignition source	S = 8 F = 1 D = 1 Total = 8	National Fire Protection Agency standoff distances for hydrogen facilities must be adhered to.
Hydrogen Transport by Pipeline	Pipeline leak with ignition source H ₂ detonation	Seismic event, collision accident, leaking fitting, etc.	S = 4 F = 1 D = 2 Total = 8	A little harder to detect unless monitors are used. Underground pipeline runs through tunnels which could trap a hydrogen cloud. Above ground structures generally protected.
			S = 5 F = 1 D = 1 Total = 5	May cause hydrogen jet fire if there is an ignition source and create overpressure. Depending on the leakage location, the fire and overpressure may or may not damage the RWGS reactor. The severity is higher than that of HTEF because RWGS has other incoming feedstock pipes such as CO2 that may increase the complexity of the plant and the damage of a hydrogen fire. Detection ranking is slightly lower because it should be easy to detect a change in incoming H ₂ line pressure.
H ₂ production	Flooding to HTEF facility, and/or damage to electrical components such as switchgear and transformers	Weather / swamp or river flooding	S = 4 F = 2 D = 1 Total = 8	Direct effect to operation is not known. But drying, cleaning the facility, and replacing components will cost money.
Syngas drying	Syngas leakage	Tank or pipe damage	S = 5 F = 1 D = 1 Total = 5	Syngas can expose plant and/or personnel to toxic and explosive hazards.
Syngas selexol separator	Syngas and selexol release	Tank or pipe damage	S = 5 $F = 1$ $D = 1$ $Total = 5$	Selexol is relatively safe since it has a low toxicity level. Prolonged skin contact may cause slight skin irritation with local redness. Syngas toxicity on the other hand is pretty high due to the carbon monoxide content. Syngas also has a fire/explosive hazard.

Table D-2. Industrial customer based FMEA results for syngas Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Industrial Plant	General Notes
Thermal energy delivery to hydrogen plant	Nuclide contamination of the process steam	Heat Exchanger Leak	S = 7 F = 1 D = 1 Total = 7	By far a more significant hazard for a BWR. Cleaning and re-starting the thermal delivery system would be required. Easily detected and stopped.
H2 storage at plant	Tank leak with ignition source H2 fire at HTEF	Tank valve or fitting leak	S = 5 F = 1 D = 1 Total = 5	Severity based on volume and pressure of tank. National Fire Protection Agency standoff distances for hydrogen facilities must be adhered to.
Multiple	H2 product loss at HTEF Kinetic energy of leaking gas	Piping or tank leak/rupture without an ignition source	S = 2 $F = 1$ $D = 1$ $Total = 2$	Depends on pressure. Pipe rupture may cause a pipe whip and impact nearby equipment and personnel. Any flow through crack is expected to be small and may disperse in atmosphere.
N/A	Damage to nearby houses, other structures, or highway	H2 detonation at HTEF	S = 2 F = 1 D = 1 Total = 2	Windows, debris, and possible injuries. Design for public safety is critical by using standoff distances and/or engineered barriers as applicable.

Table D-3. Public safety and perception based FMEA results for syngas.

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Hydrogen Transport by Truck	H2 detonation at HTEF	Fueling accident, fitting leak, valve leak, etc., along with ignition source	S = 8 F = 2 D = 1 Total = 16	Most severe hydrogen-based industrial accidents happen during fueling operations.
H2 Storage at plant	Tank rupture with ignition source H2 fire at HTEF	Forklift or other industrial equipment tears a hole in the tank. Possible high-wind missile strike.	S = 8 F = 2 D = 1 Total = 16	Severity based on volume and pressure of tank. Siting distance from public buildings needs to be sufficient or engineered barriers need to be in place.
Hydrogen Transport by Pipeline	Pipeline leak	Seismic event, collision accident, leaking fitting, etc.	S = 5 F = 1 D = 3 Total = 15	A little harder to detect unless monitors are used. Underground pipeline runs through tunnels and could trap a hydrogen cloud. Could disrupt surface roads, rail, or other underground routed services.
H2 Storage at plant	H2 detonation at HTEF	Forklift or other industrial equipment tears a hole in the tank. Possible high-wind missile strike.	S = 10 F = 1 D = 1 Total = 10	Severity based on volume and pressure of tank.
Thermal energy delivery to hydrogen plant	Nucleide contamination of the process steam	Heat Exchanger Leak	S = 10 F = 1 D = 1 Total = 10	By far a more significant hazard for a BWR. Cleaning and re-starting the thermal delivery system would be required. Easily detected and stopped. There is a very low frequency of occurrence, but negative public perception would be severe.
HTEF processes/multiple	H2 detonation at HTEF	Piping or tank leak/rupture along with an ignition source	S = 10 F = 1 D = 1 Total = 10	Siting distance from public buildings needs to be sufficient or engineered barriers need to be in place.

Table D-3. Public safety and perception based FMEA results for syngas Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
H2 production	Electrolysis stacks	High winds or tornado	S = 10	Public perception would be moderately affected.
1	damaged/toppled if		F = 1	
	stacked		D = 1	
			Total = 10	
Multiple	Damage to nearby	H2 detonation at HTEF	S = 10	Sited distance should result in minor to no damage
	houses and highway		$\mathbf{F} = 1$	but still would result in negative reaction from the
			D = 1	public.
			Total = 10	
		Syngas leakage followed by	S = 10	Severe public reaction since damage to public
		downwind deflagration in public	F = 1	property and danger to public safety. Assuming
		areas	D = 1	RWGS and syngas pipes are located at a
			Total = 10	reasonably safe distance from public areas, the
				frequency of this event is low.
Multiple	H2 fire at HTEF	Piping or tank leak/rupture along	S = 8	Sited distance should result in minor to no damage
		with an ignition source	$\mathbf{F} = 1$	but still would result in negative reaction from the
			D = 1	public.
			Total = 8	
H2 Storage at plant	H2 detonation at HTEF	Tank valve or fitting leak with	S = 8	Severity based on volume and pressure of tank.
		ignition source	$\mathbf{F} = 1$	Severity less than rupture due to plume instead of
			D = 1	cloud.
			Total = 8	
H2 Storage at plant	Tank leak with ignition	Tank valve or fitting leak	S = 8	Severity based on volume and pressure of tank.
	source		$\mathbf{F} = 1$	
	H2 fire at HTEF		D = 1	
			Total = 8	
NPP & H2 administrative	Damage to staffs' cars,	H2 detonation at HTEF	S = 8	While not directly related to NPP safety, damage
support	office buildings and		F = 1	to support buildings can affect operations and
	equipment		D = 1	negative public perception.
			Total = 8	
H2 Production	Electrolysis stacks	H2 detonation at HTEF	S = 8	Decreased credibility by public.
	damaged/toppled if		F = 1	
	stacked		D = 1	
			Total = 8	
Multiple	H2 product loss at	Piping or tank leak/rupture	S = 5	Injuries or equipment damage could result.
	HTEF	without an ignition source	F = 1	
	Kinetic energy of		D = 1	
	leaking gas		Total = 8	

Table D-3. Public safety and perception based FMEA results for syngas Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Physical protection	Damage to intrusion	H2 detonation at HTEF	S = 1	Lowered physical protection profile can lead to an
	sensors, or triggering		F = 1	opening for terrorist activity.
	multiple false alarms		D = 1	
	-		Total = 1	
Critical structure integrity	Damage to reactor	H2 detonation at HTEF	S = 0	As sited at calculated safe distance NPP to HTEF.
	building walls		F = 1	
			D = 1	
			Total = 8	
Syngas production	Syngas leakage	Damage to pipes or tanks	S = 8	While not directly related to NPP, syngas is a
			F = 2	hazardous gas so its accidental release to the
			D = 1	environment near a nuclear powerplant could
			Total = 16	receive negative public backlash.

Appendix E: FMEA Results- Methanol Synthesis Facility

The FMEA results for methanol production are listed in the following tables.

Table E-1. Nuclear power plant based FMEA results for Methanol Synthesis Facility.

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
Makeup water pipeline	Loss of makeup water supply line	Methanol detonation at Methanol facility	S = 5 F = 2 D = 1 Total = 10	Makeup water required for proper cooling, decrease could result in insufficient cooling. Assumes makeup water pipeline is buried or covered.
Spray pond	Degradation of ultimate heat sink	Methanol detonation at Methanol facility	S = 3 F = 3 D = 1 Total = 9	Debris clogging pond, possibly avoidable with proper placement. Greater frequency than methanol detonation for the makeup water pipeline since spray pond is open to atmosphere.
Cooling Tower pond	Degradation of ultimate heat sink	Methanol detonation at Methanol facility	S = 3 $F = 3$ $D = 1$ $Total = 9$	Debris clogging pond, possibly avoidable with proper placement. Greater frequency than methanol detonation for the makeup water pipeline since spray pond is open to atmosphere.
Forced air cooling for non- safety buildings	Loss of HVAC equipment	Methanol detonation at Methanol facility	S = 2 F = 3 D = 1 Total = 6	Buildings with non-safety critical systems nor reactor building.
NPP & Methanol Facility administrative support	Damage to staffs' cars, office buildings and equipment	Methanol detonation at Methanol facility	S = 2 F = 3 D = 1 Total = 6	While not directly related to NPP safety, damage to support buildings can affect operations.
External Power	Loss of offsite power	Methanol detonation at Methanol facility	S = 1 F = 1 D = 1 Total = 1	Depends on placement. Assume sited at safe distance.

Table E-1. Nuclear power plant based FMEA results for Methanol Synthesis Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
Primary loop transport of process steam	Pipe Rupture after MSIV, damage to turbine building equipment, possibly safety power buses, depending on the plant	Operational vibration seismic, and erosion	S = 1 $F = 1$ $D = 1$ $Total = 1$	Depends on placement. Assume sited at safe distance.
Non-Safety Service water pump house	Damage and/or loss of service water building and equipment	Methanol detonation at Methanol facility	S = 1 F = 1 D = 1 Total = 1	Depends on placement. Assume sited at safe distance.
External Supply Tanks integrity	Damage to CST, other supply tanks	Methanol detonation at Methanol facility	S = 1 F = 1 D = 1 Total = 1	Depends on placement. Assume sited at safe distance.
Physical protection	Damage to intrusion sensors, or triggering multiple false alarms	Methanol detonation at Methanol facility	S = 1 F = 1 D = 1 Total = 1	Lowered physical protection profile can lead to an opening for terrorist activity. Assume sited at safe distance.

Table E-2. Industrial customer based FMEA results for Methanol Synthesis Facility.

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
H2 feedstock transport by pipeline	Leakage during delivery	Fueling accident, toxicity	S = 4 F = 2 D = 2 Total = 16	A little harder to detect unless monitors are used. Underground pipeline runs through tunnels could trap a hydrogen cloud.
Methanol fixed bed synthesis reactor	Increased heat then pressure for reactor, detonation of methanol	Fouling in shell and reduced heat transfer	S = 8 F = 1 D = 2 Total = 16	Reduced heat transfer in reactor could lead to temperature increase and therefore pressure increase and possible detonation.
Multiple	Methanol detonation at Methanol Facility	Piping, reactor, or distillation column leak/rupture along with an ignition source	S = 10 $F = 1$ $D = 1$ $Total = 10$	Depends on location of break in system for concentration of methanol or other chemicals. Assume sited at safe distance.
Methanol fixed bed synthesis reactor	Methanol detonation at Methanol Facility	Runaway reaction/methanation and failed rupture disk or safety release valve	S = 10 F = 1 D = 1 Total = 10	Rapid increase in pressure and temperature without proper release could lead to severe detonation. Assume sited at safe distance.
Multiple	Methanol fire at Methanol Facility	Piping, reactor, or distillation column leak/rupture along with an ignition source	S = 8 F = 1 D = 1 Total = 8	Depends on location of break in system for concentration of methanol or other chemicals. Assume sited at safe distance.
Methanol fixed bed synthesis reactor	Methanation- Increased heat then pressure for reactor, explosion	Loss of cooling water, high CO concentration, presence of oxygen	S = 8 F = 1 D = 1 Total = 8	Cooling water required to keep reaction at constant temperature, increase in temperature due to less cooling could result in increased pressure and possibly detonation.
Multiple	Methanol product loss at Methanol synthesis facility, kinetic energy of leaking gas	Piping, reactor, or distillation column leak/rupture along with an ignition source	S = 2 F = 1 D = 1 Total = 2	Depends on pressure for magnitude of leak/rupture.

Table E-2. Industrial customer based FMEA results for Methanol Synthesis Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
RWGS	Explosion in normal operation	Lack of fuel gas in network causing an accumulation, high pressure in fuel gas network, large variation in fuel gas density outside burner operating window, lack of combustion air, blockage of air intake, positive relative pressure in radiation zone	S = 8 F = 1 D = 1 Total = 8	Based on Total Energies report on Major Risk Scenarios and Safety & Environment Barriers for Steam Crackers. Severity of assets are complete destruction so high severity of hazard. Easy detection since multiple alarms and trips based on sensors and control systems to prevent the mechanisms of failure.
RWGS	Explosion in radiation section during start up	Accumulation of fuel gas due to leak or failure of ignitor	S = 8 $F = 1$ $D = 1$ $Total = 8$	Mitigation is mostly in proper execution of furnace start-up procedure.
RWGS	Radiation tube rupture which leads to fire in vicinity of furnace	furnace trip while tubes have high coke content, low hydrocarbon supply flow rate, low dilution steam flow, tube or welding defect, thermal degradation of tube, thermal shock with introduction of cold feedstock, cold naphtha entry in dilution steam	S = 5 F = 4 D = 2 Total = 40	Common event mostly due to trips, temperature alarm and operator action to partial trip, CO monitoring with a partial trip, and periodic inspection can mitigate. Severity is moderate due to localized nature of the break.
CO ₂ capture by selexol solvent	Decreased capture efficiency of CO2, decreased MeOH synthesis, overpressure at outlet	Insufficient refrigeration of selexol solvent	S = 2 $F = 2$ $D = 1$ $Total = 4$	Assumes significant decrease in inefficiency of the selexol at capturing the CO2 and how off optimal it makes the ratio of H ₂ :CO.

Table E-3. Public safety and perception based FMEA results for Methanol Synthesis Facility.

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
NA	Damage to nearby houses, public buildings, and highway	Methanol detonation at Methanol facility	S = 8 F = 2 D = 1 Total = 16	Severe public reaction since damage to public property and danger to public safety.
NPP & Methanol Facility administrative support	Damage to staffs' cars, office buildings and equipment	Methanol detonation at Methanol facility	S = 7 F = 2 D = 1 Total = 14	While not directly related to NPP safety, damage to support buildings and staff within would trigger severe public reaction.
H ₂ feedstock transport by pipeline	Leakage during delivery	Fueling accident, toxicity	S = 7 F = 2 D = 1 Total = 14	Severe public reaction if externally visible/exposed structure or pipelines.
Physical protection	Damage to intrusion sensors, or triggering multiple false alarms	Methanol detonation at Methanol facility	S = 3 F = 3 D = 1 Total = 9	Lowered physical protection profile can lead to an opening for terrorist activity.

Table E-4. Economy based FMEA results for Methanol Synthesis Facility.

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
NA	Damage to nearby houses, public buildings, and highway	Methanol detonation at Methanol facility	S = 8 F = 3 D = 1 Total = 24	Depends on placement. Liability- responsibility for damage repair.
NPP & Methanol Facility administrative support	Damage to staffs' cars, office buildings and equipment	Methanol detonation at Methanol facility	S = 6 F = 3 D = 1 Total = 18	While not directly related to NPP safety, damage to support buildings can affect operations.
Non-Safety Service water pump house	Damage and/or loss of service water building and equipment	Methanol detonation at Methanol facility	S = 5 F = 3 D = 1 Total = 15	Depends on placement.
Forced air cooling for non-safety buildings	Loss of HVAC equipment	Methanol detonation at Methanol facility	S = 5 F = 3 D = 1 Total = 15	Buildings with non-safety critical systems nor reactor building.
Makeup water pipeline	Loss of makeup water supply line	Methanol detonation at Methanol facility	S = 3 F = 3 D = 1 Total = 9	Makeup water required for NPP safety, cooling system, would require immediate attention and pause of normal operations.
Spray pond	Degradation of ultimate heat sink	Methanol detonation at Methanol facility	S = 3 F = 3 D = 1 Total = 9	Debris clogging pond, possibly avoidable with proper placement.
Cooling Tower pond	Degradation of ultimate heat sink	Methanol detonation at Methanol facility	S = 3 F = 3 D = 1 Total = 9	Debris clogging pond, possibly avoidable with proper placement.

Table E-4. Economy based FMEA results for Methanol Synthesis Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
External Power	Loss of offsite power	Methanol detonation at Methanol facility	S = 1 F = 3 D = 1 Total = 3	Dependent on emergency power system, how long emergency power is required. Assume safe siting distance.
Primary loop transport of process steam	Pipe Rupture after MSIV Damage to turbine building equipment, possibly safety power buses, depending on the plant	Operational vibration seismic, and erosion	S = 1 F = 3 D = 1 Total = 3	Primary loop is essential for heat source of power cycle. No power generation for NPP leads to no basic commodity generation. Assume safe siting distance.
Physical protection	Damage to intrusion sensors, or triggering multiple false alarms	Methanol detonation at Methanol facility	S = 1 F = 3 D = 1 Total = 3	Lowered physical protection profile can lead to an opening for terrorist activity. Assume safe siting distance.
External Supply Tanks integrity	Damage to CST, other supply tanks	Methanol detonation at Methanol facility	S = 1 $F = 3$ $D = 1$ $Total = 3$	Depends on placement. Assume safe siting distance.
CO2 capture by selexol solvent	Decreased capture efficiency of CO2, decreased MeOH synthesis	Insufficient refrigeration of selexol solvent	S = 2 F = 2 D = 1 Total = 4	Depends how inefficient the selexol becomes at capturing the CO2 and how off optimal it makes the ratio of H ₂ :CO.

Page intentionally left blank.

Appendix F: FMEA Results- Petroleum Refinery Facility

The FMEA results for a petroleum refinery facility are listed in the following tables.

Table F-1. Nuclear power plant based FMEA results for Petroleum Refinery Facility.

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
Water contamination	toxic, settles in water, low places,	leaks of refinery products (e.g., H2S) to the water system in NPP	S = 5 $F = 3$ $D = 2$ $Total = 30$	Potentially hindered shutdown and equipment damage for NPP. The control room environmental filtering needs to be capable of protecting the room from all potential customer hazards.
NPP & Refinery administrative support	Damage to staffs' cars, office buildings and equipment	Oil/byproducts detonation at Refinery plant	S = 6 F = 2 D = 1 Total = 12	NPP operations hindered until repairs are made.
Water contamination	Staff health threat	Contamination by the spill of the Refinery products or feedstocks	S = 6 F = 2 D = 1 Total = 12	Hindered operation of the NPP.
External Power	Loss of offsite power	Oil/byproducts detonation at Refinery facility. Flares can activate, lots of heat within the power plant	S = 5 F = 2 D = 1 Total = 10	Offsite power loss severity is variable, depending on the reactor design. Safe siting distance using protective barriers where necessary screen this out in a deterministic assessment.
External Supply Tanks integrity	Damage to CST, other supply tanks	Oil/byproducts detonation at Refinery plant	S = 5 F = 2 D = 1 Total = 10	Potentially hindered shutdown. NPP would remain offline until tank farm is repaired.
Water contamination	pH change in intake water	Contamination by the spill of the Refinery products or feedstocks	S = 5 F = 2 D = 1 Total = 10	Need to shut down the NPP unexpectedly due to possible damage to pumps and other equipment.

Table F-1. Nuclear power plant based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
Forced air cooling for non- safety buildings	Loss of HVAC equipment	Oil/byproducts detonation at Refinery plant	S = 4 $F = 2$ $D = 1$ $Total = 8$	NPP operations hindered until repairs are made.
Non-Safety Service water pump house	Damage and/or loss of service water building and equipment	Oil/byproducts detonation at Refinery plant	S = 4 F = 2 D = 1 Total = 8	NPP would need to shut down safely until repairs are made.
Water contamination	Clogging of water intake screens	Contamination by the spill of the Refinery products or feedstocks	S = 4 $F = 2$ $D = 1$ $Total = 8$	Need to shut down the NPP unexpectedly due to clogged intakes.
Physical protection	Damage to intrusion sensors, or triggering multiple false alarms	Oil/byproducts detonation at Refinery plant	S = 3 $F = 2$ $D = 1$ $Total = 6$	Lowered security posture. Impacted security but not directly affecting the nuclear safety.
Primary loop transport of process steam	Pipe Rupture after MSIV Damage to turbine building equipment, possibly safety power buses, depending on the plant	Operational vibration due to the detonation or explosion from the refinery plant	S = 3 F = 2 D = 1 Total = 6	More likely to affect the piping outside of the reboiler room leading to the customer. Prompt loss of heat load would occur.
Spray pond	Degradation of ultimate heat sink	Oil/byproducts detonation at Refinery plant fills the spray pond with debris	S = 3 F = 2 D = 1 Total = 6	NPP may have to shut down, depending on the severity of the debris.
Primary loop transport of process steam	Pipe Rupture after MSIV Damage to turbine building equipment, possibly safety power	Corrosion due to chemical release from the refinery plant	S = 1 $F = 2$ $D = 2$ $Total = 4$	Corrosives would take some time to affect the piping. Regular inspection could detect and prevent the problem.

Table F-1. Nuclear power plant based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
	buses, depending on the plant			
Makeup water pipeline	Loss of makeup water supply line	Oil/byproducts detonation at Refinery plant	$\begin{aligned} S &= 1 \\ F &= 2 \\ D &= 1 \\ \textbf{Total} &= 2 \end{aligned}$	Would not affect NPP operation. Customer revenue would be lost until repaired.
Spent fuel storage (dry)	Damage to casks causes radiation leak	Oil/byproducts detonation at Refinery plant	S = 1 F = 2 D = 1 Total = 2	Dry casks are rated for fire protection.
Steam diversion load roughly 5% thermal	Prompt loss of thermal load	Oil/byproducts detonation at Refinery plant	S = 1 $F = 2$ $D = 1$ $Total = 2$	Not an issue unless the thermal diversion exceeds 30%.

Table F-2. Industrial customer based FMEA results for Petroleum Refinery Facility.

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
Hydrocarbon production and storage	hazardous chemical release	Leak of NAPTHA (pentane + hexane), (different compounds to catalytic reformer for gasoline)	S = 4 F = 4 D = 3 Total =48	Toxicity varies from 636 mg/kg to 25000 mg/kg depending on the compositions.
Desalting	Toxic BOC release	leakage of the toxic chemicals/Corrosion of the pipelines	S = 9 F = 1 D = 5 Total =45	Hypothetical events based on the physical understanding of the process.
Upgrading and Conversion	Fire	Ignition of a buildup of flammable vapors	S = 10 F = 4 D = 1 Total =40	This is based on two actual events reported in 2005. An overflowed flammable vapor cloud flowing down to the ground ignited with an idling diesel pickup truck present during the start-up of a raffinate splitter tower. 15 workers killed, 180 others injured. \$21.1 billion settlement for the victims and their families. The other events happened in a distillation tower in 2006. 11 workers killed and 17 others injured. \$20 million settlement for the victims and their families.
H2S storage/use at plant/Contaminant Removal	Toxic H2S release	Leakage of the H2S from storage tanks, pipes	S = 10 F = 1 D = 4 Total = 40	Hypothetical events based on the physical understanding of the process.
Hydrocarbon production and storage	High temperature shift converter uses CO and H to create the syngas (used in methane reforming process after CO is created). Issue is carbon oxides need separated to get CO.	Leak of Carbon Monoxide	S = 4 F = 2 D = 4 Total =32	Carbon monoxide leakage is hard to detect, and a good amount of inhalation (3760 ppm) will cause acute toxicity.

Table F-2. Industrial customer based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
Maintenance	Fire	Ignition of gasoline components; ignition of Naphtha. The root cause of these events come from the human error.	S = 7 F = 4 D = 1 Total =28	This is based on two actual events reported in CSB. One happened in 1999 during a pipe removal which transports Napthta. Several attempts fail to drain the Naptha lines. Four workers killed and one critically injured. The other happened in 2004 when ignition happened from gasoline components release during maintenance. The works fails to identify a open valve that needed to be closed. 4 workers were seriously injured. Over \$13 million in property damage.
Sulfuric Acid	Corrosive, can cause leaks in pipes with worse consequences	integrity failure	S = 6 F = 2 D = 2 Total =24	Lamont refinery accident - destroyed (launched) tower.
Upgrading and Conversion	Explosions and fires	corrosion of vapor pipeline	S = 10 F = 2 D = 1 Total =20	This is a real accident that happened in Shell plant explosion in Norco, Louisiana in 1988. Seven Shell workers were killed during the explosion and 48 residents and Shell workers were injured in the explosion. The explosion released 159 million pounds (72 kt) of toxic chemicals into the air, which led to widespread damage and the evacuating of 4,500 people.
Upgrading and Conversion	Explosions and Fire; toxic hydrofluoric acid release	rupture of a steel piping component with high nickel and copper content that had corroded from HF and thinned faster than adjacent piping components with lower nickel and copper content.	S = 10 F = 2 D = 1 Total =20	This is based on an actual event reported in 2019 at Philadelphia Energy Solutions (PES) refinery in Philadelphia, Pennsylvania. The refinery announced it would shut down operations the same month, and filed for bankruptcy a month later. PES estimated that 5,239 pounds of HF released from piping and equipment during the incident. It estimated that 1,968 pounds of the released HF was

Table F-2. Industrial customer based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
				contained by water spray within the unit and was processed in the refinery wastewater treatment plant, and that 3,271 pounds of HF was released to the atmosphere and was not contained by water spray. A PES also estimated that about 676,000 pounds of hydrocarbons were released during the event, of which an estimated 608,000 pounds were combusted. Marsh JLT Specialty reported that the incident resulted in an estimated
Upgrading and Conversion	Fire; toxic chlorine release	temperature control failure; propane vapor release from cracked control station piping	S = 9 F = 2 D = 1 Total = 18	property damage loss of \$750 million. This is based on an actual event reported in 2007. Four workers injured were seriously burned, including a contractor. The refinery was completely shut down for just under two months and operated at reduced capacity for nearly a year. The nearby chlorine container was affected, and 2.5 tons of chlorine has been released. Direct losses attributed to the fire were reported to exceed \$50 million
Upgrading and Conversion	Explosions and Fire	Heat exchanger rupture due to high temperature hydrogen attack	S = 8 F = 2 D = 1 Total =16	This is based on an actual event reported in 2010. Hydrogen and naphtha at more than 500°F were released. SEVEN FATALITIES were reported. Moderate property damage from \$500,000 to \$2 million.
Hydrocarbon production and storage	hazardous chemical release	Leak of alkylation	S = 5 $F = 1$ $D = 3$ $Total = 15$	alkylating agents are highly toxic to mucosal cells resulting in oral mucosal ulceration and effects on the intestinal mucosa.
Maintenance	Explosions and Fire	inadvertently directing air inside the regenerator through the reactor and the main column, then into the	S = 7 F = 2 D = 1 Total =14	This is based on an actual event reported in 2018. Shaking within a mile away. 100 metal fragments propelled (~1200 feet) within the operating areas. Exploration debris punctured

Table F-2. Industrial customer based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
		gas concentration unit. Failure to control the air flow occurred during the shutdown. Husky Superior Refinery did not effectively implement process safety management systems		a asphalt tank, spelling out. The city evacuated 2507 residents within 2 miles north, 3miles to east and west and 10 miles south of the refinery. 36 refinery and contract workers injured (11 of them suffered from OSHA recordable injuries). This incident resulted in \$550 million of on-site and \$110,000 of offsite property damage.
Fractionation	Explosions and Fire	Pipe rupture	S = 6 F = 2 D = 1 Total =12	This is based on an actual event reported in 2012. Pipe rupture of light gas oil produced a vapor cloud that caught fire, and also enabled the release of flammable, toxic vapor. Approximately 15000 people from the surrounding area sought medical treatment due to a large plume of particulates and vapor traveling across the area
Maintenance	Fire	Operation error	S = 6 $F = 2$ $D = 1$ $Total = 12$	This is based on an actual event reported in 2016. 4 workers and two others seriously injured
Fractionation	Channel Clogged	Buildup of the materials inside the channel	S = 4 F = 1 D = 3 Total =12	Hypothetical events based on the physical understanding of the process.
Oil storage at plant	potential chemical releases	leaks of the oil	S = 4 F = 1 D = 3 Total =12	Hypothetical events based on the physical understanding of the process.
CO, used as fuel and also as byproduct	Poisonous, asphyxiant	Byproduct of RWGS, also incomplete combustion	S = 3 F = 1 D = 4 Total =12	Hypothetical events based on the physical understanding of the process.

Table F-2. Industrial customer based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
Benzene	Cancerous,	integrity failure	S = 4 F = 1 D = 3 Total =12	Hypothetical events based on the physical understanding of the process.
HF feedstock	Acidic, bone-seeker	integrity failure	S = 6 F = 1 D = 2 Total = 12	Hypothetical events based on the physical understanding of the process.
Hydrocarbon production and storage	hazardous chemical release	Leak of propylene	S = 4 $F = 1$ $D = 3$ $Total = 12$	An osmolar gap >10 mmoles/L suggests that the serum propylene glycol concentration is high enough to cause toxicity
Purging	Fire	Flammable gas leaks from a failed separation vessel where over-pressured happened and no safety mitigation system is available.	S = 5 F = 2 D = 1 Total = 10	This is a real accident that happened in Sonat Exploration Company in 1988. The fire results in the damage of the separator, piping, personal vehicles, backhoe, oil and water storage tanks, which terminates the operation of the refinery. 4 workers killed and significant damage to facility. In addition to the fatalities, the incident resulted in about \$200,000 worth of damage, including the destruction of the third-stage separator, four private vehicles, and a backhoe and damage to the facility storage tanks
Upgrading and Conversion	Explosions and Fire	Operation error	S = 5 $F = 2$ $D = 1$ $Total = 10$	This is based on an actual event reported in 2015 at ExxonMobil Torrance Refinery. The accidents severely damaged the "electrostatic precipitator" and four contract worker were injured. A tank close to the electrostatic precipitator containing HF, water, hydrocarbons, and chemical additives was hit.

Table F-2. Industrial customer based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
Upgrading and Conversion	Explosions and Fire	Operations error, human factors.	S = 5 F = 2 D = 1 Total = 10	This is based on an actual events reported in 2022. Naphtha filled a fuel gas mix drum that was normally only for vapors, and a flammable naphtha vapor cloud on the ground eventually ignited.
CO2 feedstock	Asphyxiant	leaks of the CO2	S = 2 F = 1 D = 5 Total = 10	Hypothetical events based on the physical understanding of the process.
Hydrocarbon production and storage	Mechanical injuries, can cut, create missiles, bend pipe, etc.	High-pressure steam leak	S = 5 F = 2 D = 1 Total = 10	The high-pressure steam may cause damage to the facilities and the surrounding staffs working in the refinery.
Hydrocarbon production and storage	hazardous chemical release	Leak of propane	S = 3 $F = 1$ $D = 3$ $Total = 9$	It has been reported that brief inhalation exposures to 10,000 ppm propane cause no symptoms in humans
Hydrocarbon production and storage	hazardous chemical release	Leak of iso-butane	S = 3 $F = 1$ $D = 3$ $Total = 9$	Acute oral toxicity: LD50: > 5,000 mg/kg; Acute inhalation toxicity: LC50: > 31 mg/l
Hydrocarbon production and storage	hazardous chemical release	Leak of jet fuel	S = 3 F = 1 D = 3 Total =9	Acute oral toxicity: LD50: > 2,000 mg/kg;
Hydrocarbon production and storage	hazardous chemical release	Leak of diesel fuel	S = 3 $F = 1$ $D = 3$ $Total = 9$	Acute oral toxicity: LD50: > 5,000 mg/kg; Leak of Diesel is also a blend. Lowers the sulfur and aromatics (black soot).
Hydrocarbon production and storage	hazardous chemical release	Leak of heptane and cyclo- hexane byproduct	S = 3 $F = 1$ $D = 3$ $Total = 9$	Acute oral toxicity: LD50: > 5,000 mg/kg;

Table F-2. Industrial customer based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
Upgrading and Conversion	Explosions and Fire	Pipe rupture	S = 4 F = 2 D = 1 Total =8	This is based on an actual event reported in 2009. A pipe rupture causes an explosion, leading to the damage of a light structural elements. Two refinery operators and two contractors suffered serious burns
Desalting	Fire	detonation for hydrocarbons	S = 8 $F = 1$ $D = 1$ $Total = 8$	Hypothetical events based on the physical understanding of the process.
Stream Quality Improvement and Blending	Fire	Buildup of the flammable vapors	S = 8 $F = 1$ $D = 1$ $Total = 8$	Hypothetical events based on the physical understanding of the process.
NAPTHA storage at plant	Fires	leaks of the NAPTHA	S = 8 F = 1 D = 1 Total = 8	Hypothetical events based on the physical understanding of the process.
Oil storage at plant	Fires	leaks of the oil	S = 8 F = 1 D = 1 Total = 8	Hypothetical events based on the physical understanding of the process.
Upgrading and Conversion	Explosions and fires	under investigation	S = 3 F = 2 D = 1 Total =6	This is a real accident that happened in Shell Oil refinery in 1989. The fire burns out for three hours. Two Shell contract employees were injured. Neighborhoods were not being evacuated.
Upgrading and Conversion	Fire	valve leakage	S = 3 F = 2 D = 1 Total =6	This is based on an actual event reported in 2015 at Delaware City Refinery. The fire burned one hour before isolation.

Table F-2. Industrial customer based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
Desalting	Internal flooding	Disposal water leakage	S = 2 F = 1 D = 3 Total =6	Hypothetical events based on the physical understanding of the process.
Feedstock (crude oil) Transport by Truck	Fueling accident, toxic chemical release	Leakage during delivery	S = 6 F = 1 D = 1 Total =6	Hypothetical events based on the physical understanding of the process.
Hydrocarbon production and storage	hazardous chemical release	Leak of butane	S = 2 F = 1 D = 3 Total =6	IDLH value: 1,600 ppm
NAPTHA storage at plant	potential chemical releases (toxicity)	leaks of the NAPTHA	S = 4 F = 1 D = 1 Total =4	Hypothetical events based on the physical understanding of the process.
Hydrocarbon production and storage	hazardous chemical release	Leak of sour water	S = 2 F = 1 D = 2 Total =4	There can be impacts of drinking water or minor damage of the facilities
Hydrocarbon production and storage	hazardous chemical release	Leak of methane	S = 1 F = 1 D = 3 Total =3	Methane is non-toxic
Hydrocarbon production and storage	hazardous chemical release	Leak of ethane	S = 1 F = 1 D = 3 Total =3	Ethane is non-toxic
Hydrocarbon production and storage	hazardous chemical release	Leak of butylene	S = 1 F = 1 D = 3 Total =3	limited toxicity

Table F-2. Industrial customer based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Methanol	General Notes
Hydrocarbon production and storage	hazardous chemical release	Leak of iso-butylene	S = 1 F = 1 D = 3 Total =3	limited toxicity
Hydrocarbon production and storage	hazardous chemical release	Leak of vacuum gasoil (BP 700F to 1000F)	S = 1 F = 1 D = 3 Total =3	No datasheet for gasoil. Leak of gasoline is a blend of all the different streams to make the final product.
Hydrocarbon production and storage	hazardous chemical release	Leak of asphalt (can crack it into some other streams, will auto-ignite)	S = 1 F = 1 D = 3 Total =3	No data available for oral acute toxicity
Hydrocarbon production and storage	hazardous chemical release	Leak of coke for burning	S = 1 F = 1 D = 3 Total =3	No data available for oral acute toxicity
Desalting	Failures of removing residual water	pumps malfunctions; unsuccessful splits	S = 1 F = 1 D = 3 Total =2	Hypothetical events based on the physical understanding of the process.

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility.

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Hydrocarbon production and storage	hazardous chemical release	Leak of NAPTHA (pentane + hexane), (different compounds to catalytic reformer for gasoline)	S = 5 F = 4 D = 3 Total = 60	public concern due to the toxic chemical release
Water contamination	toxic, settles in water, low places	leaks of refinery products (e.g., H2S) to the water system in NPP	S = 8 F = 3 D = 2 Total = 48	Significant public concern will arise when it is announced that the water around the plant is contaminated.
Hydrocarbon production and storage	High temperature shift converter uses CO and H to create the syngas (used in methane reforming process after CO is created). Issue is carbon oxides need separated to get CO	Leak of Carbon Monoxide	S = 4 F = 2 D = 4 Total = 32	public concern due to the toxic chemical release (less toxicity compared to H2S)
Desalting	Toxic BOC release	leakage of the toxic chemicals/Corrosion of the pipelines	S = 5 F = 1 D = 5 Total = 25	public concern due to the toxic chemical release
H2S storage/use at plant/Contaminant Removal	Toxic H2S release	Leakage of the H2S from storage tanks, pipes	S = 5 $F = 1$ $D = 4$ $Total = 20$	public concern due to the toxic chemical release
H2S storage/use at plant/Contaminant Removal	Toxic H2S release	Leakage of the H2S from storage tanks, pipes	S = 5 F = 1 D = 4 Total = 20	public concern due to the toxic chemical release
Water contamination	Staff health threat	Contamination by the spill of the Refinery products or feedstocks	S = 8 F = 2 D = 1 Total = 16	Significant public concern will arise when it is announced that the water around the plant is contaminated.

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Water contamination	pH change in intake water	Contamination by the spill of the Refinery products or feedstocks	S = 8 F = 2 D = 1 Total = 16	Significant public concern will arise when it is announced that the water around the plant is contaminated.
CO, used as fuel and also as byproduct	Poisonous, asphyxiant	Byproduct of RWGS, also incomplete combustion	S = 4 F = 1 D = 4 Total = 16	public concern due to the toxic chemical release (less toxicity compared to H2S)
N/A	Damage to nearby houses, public buildings, and highway	Oil/byproducts detonation at Refinery plant	S = 8 F = 2 D = 1 Total = 16	Severe public reaction since damage to public property and danger to public safety.
Oil storage at plant	potential chemical releases	leaks of the oil	S = 5 F = 1 D = 3 Total = 15	public concern due to the toxic chemical release
Hydrocarbon production and storage	hazardous chemical release	Leak of propylene	S = 5 F = 1 D = 3 Total = 15	public concern due to the toxic chemical release
CO2 feedstock	Asphyxiant	leaks of the CO2	S = 3 F = 1 D = 5 Total = 15	public concern due to the toxic chemical release (less toxicity compared to H2S)
Hydrocarbon production and storage	hazardous chemical release	Leak of propane	S = 5 F = 1 D = 3 Total = 15	public concern due to the toxic chemical release
Hydrocarbon production and storage	hazardous chemical release	Leak of iso-butane	S = 5 F = 1 D = 3 Total = 15	public concern due to the toxic chemical release

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Hydrocarbon production and storage	hazardous chemical release	Leak of jet fuel	S = 5 F = 1 D = 3 Total = 15	public concern due to the toxic chemical release
Hydrocarbon production and storage	hazardous chemical release	Leak of diesel fuel	S = 5 $F = 1$ $D = 3$ $Total = 15$	public concern due to the toxic chemical release
Hydrocarbon production and storage	hazardous chemical release	Leak of heptane and cyclo- hexane byproduct	S = 5 F = 1 D = 3 Total = 15	public concern due to the toxic chemical release
NPP & Refinery administrative support	Damage to staffs' cars, office buildings and equipment	Oil/byproducts detonation at Refinery plant	S = 7 F = 2 D = 1 Total = 14	While not directly related to NPP safety, damage to support buildings and staff within would trigger severe public reaction.
Primary loop transport of process steam	Pipe Rupture after MSIV, Damage to turbine building equipment, possibly safety power buses, depending on the plant	Corrosion due to chemical release from the refinery plant	S = 3 F = 2 D = 2 Total = 12	raise public concern related to safety issue in NPP but less than the detonation.
Spent fuel storage (dry)	Damage to casks causes radiation leak	Oil/byproducts detonation at Refinery plant	S = 6 $F = 2$ $D = 1$ $Total = 12$	raise public concern regarding the large release of the radiation to the environment
Hydrocarbon production and storage	hazardous chemical release	Leak of alkylation	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Oil storage at plant	potential chemical releases	leaks of the oil	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)
Benzene	Cancerous,	integrity failure	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)
Hydrocarbon production and storage	hazardous chemical release	Leak of butane	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)
Hydrocarbon production and storage	hazardous chemical release	Leak of methane	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)
Hydrocarbon production and storage	hazardous chemical release	Leak of ethane	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)
Hydrocarbon production and storage	hazardous chemical release	Leak of butylene	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)
Hydrocarbon production and storage	hazardous chemical release	Leak of iso-butylene	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)
Hydrocarbon production and storage	hazardous chemical release	Leak of vacuum gasoil (BP 700F to 1000F)	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Hydrocarbon production and storage	hazardous chemical release	Leak of asphalt (can crack it into some other streams, will autoignite)	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)
Hydrocarbon production and storage	hazardous chemical release	Leak of coke for burning	S = 4 F = 1 D = 3 Total = 12	public concern due to the toxic chemical release (less toxicity compared to H2S)
Primary loop transport of process steam	Pipe Rupture after MSIV. Damage to turbine building equipment, possibly safety power buses, depending on the plant	Operational vibration due to the detonation or explosion from the refinery plant	S = 5 $F = 2$ $D = 1$ $Total = 10$	raise public concern related to safety issue in NPP
Upgrading and Conversion	Explosions and Fire; toxic HF release	rupture of a steel piping component with high nickel and copper content that had corroded from HF and thinned faster than adjacent piping components with lower nickel and copper content	S = 5 F = 2 D = 1 Total = 10	public concern due to the toxic chemical release
Upgrading and Conversion	Fire; toxic chlorine release	temperature control failure; propane vapor release from cracked control station piping	S = 5 $F = 2$ $D = 1$ $Total = 10$	public concern due to the toxic chemical release
HF feedstock	Acidic, bone-seeker	integrity failure	S = 5 $F = 2$ $D = 1$ $Total = 10$	public concern due to the toxic chemical release
Water contamination	Clogging of water intake screens	Contamination by the spill of the Refinery products or feedstocks	S = 4 $F = 2$ $D = 1$ $Total = 8$	less concern compared to the chemistry contamination

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Upgrading and Conversion	Fire	Ignition of a buildup of flammable vapors	S = 2 F = 4 D = 1 Total = 8	public concern due to the potential impacts on the air quality
Maintenance	Fire	Ignition of gasoline components; ignition of Naphtha; the root cause of these events come from the human error	S = 2 F = 4 D = 1 Total = 8	public concern due to the potential impacts on the air quality
Physical protection	Damage to intrusion sensors, or triggering multiple false alarms	Oil/byproducts detonation at Refinery plant	S = 3 F = 2 D = 1 Total = 6	Lowered physical protection profile can lead to an opening for terrorist activity.
Feedstock (crude oil) Transport by Truck	Fueling accident, toxic chemical release	Leakage during delivery	S = 6 F = 1 D = 1 Total = 6	raise public reaction if visible explosion can be seen on the road.
Hydrocarbon production and storage	hazardous chemical release	Leak of sour water	S = 3 $F = 1$ $D = 2$ $Total = 6$	public concern due to the toxic chemical release (less toxicity compared to H2S)
NAPTHA storage at plant	potential chemical releases (toxicity)	leaks of the NAPTHA	S = 5 $F = 1$ $D = 1$ $Total = 5$	public concern due to the toxic chemical release
Feedstock (crude oil) Transport by Truck	Fueling accident, toxic chemical release	Leakage during delivery	S = 5 $F = 1$ $D = 1$ $Total = 5$	public concern due to the toxic chemical release
NAPTHA storage at plant	potential chemical releases (toxicity)	leaks of the NAPTHA	S = 5 $F = 1$ $D = 1$ $Total = 5$	public concern due to the toxic chemical release

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Critical structure integrity	Damage to critical structures	Oil/byproducts detonation at Refinery plant	S = 2 F = 2 D = 1 Total = 4	The failure of the structure integrity may raise a public concern.
External Power	Loss of offsite power	Oil/byproducts detonation at Refinery facility. Flares can activate, lots of heat within the power plant	S = 2 F = 2 D = 1 Total = 4	The LOOP may raise a public concern that is worse than the others.
Makeup water pipeline	Loss of makeup water supply line	Oil/byproducts detonation at Refinery plant	S = 2 F = 2 D = 1 Total = 4	raise limited public concern
Sulfuric Acid	Corrosive, can cause leaks in pipes with worse consequences	integrity failure	S = 1 $F = 2$ $D = 2$ $Total = 4$	limited public concern if this happen inside the refinery plant
Upgrading and Conversion	Explosions and fires	corrosion of vapor pipeline	S = 2 F = 2 D = 1 Total = 4	public concern due to the potential impacts on the air quality
Upgrading and Conversion	Explosions and Fire	Heat exchanger rupture due to high temperature hydrogen attack	S = 2 F = 2 D = 1 Total = 4	public concern due to the potential impacts on the air quality
Maintenance	Explosions and Fire	inadvertently directing air inside the regenerator through the reactor and the main column, then into the gas concentration unit. Failure to control the air flow occurred during the shutdown. Husky Superior Refinery did not effectively	S = 2 F = 2 D = 1 Total = 4	public concern due to the potential impacts on the air quality

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
		implement process safety management systems		
Fractionation	Explosions and Fire	Pipe rupture	S = 2 F = 2 D = 1 Total = 4	public concern due to the potential impacts on the air quality
Maintenance	Fire	Operation error	S = 2 F = 2 D = 1 Total = 4	public concern due to the potential impacts on the air quality
Purging	Fire	Flammable gas leaks from a failed separation vessel where overpressurization happened and no safety mitigation system is available	S = 2 F = 2 D = 1 Total = 4	public concern due to the potential impacts on the air quality
Upgrading and Conversion	Explosions and Fire	Operation error	S = 2 F = 2 D = 1 Total = 4	public concern due to the potential impacts on the air quality
Upgrading and Conversion	Explosions and Fire	Operations error, human factors	S = 2 F = 2 D = 1 Total = 4	public concern due to the potential impacts on the air quality
Upgrading and Conversion	Explosions and Fire	Pipe rupture	S = 2 $F = 2$ $D = 1$ $Total = 4$	public concern due to the potential impacts on the air quality
Upgrading and Conversion	Explosions and fires	under investigation	S = 2 F = 2 D = 1 Total = 4	public concern due to the potential impacts on the air quality

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Upgrading and Conversion	Fire	valve leakage	S = 2 F = 2 D = 1 Total = 4	public concern due to the potential impacts on the air quality
Fractionation	Channel Clogged	Buildup of the materials inside the channel	S = 1 F = 1 D = 3 Total = 3	Given that the shutdown of refinery is safe enough. No specific concern will arise
Fractionation	Channel Clogged	Buildup of the materials inside the channel	S = 1 F = 1 D = 3 Total = 3	Given that the shutdown of refinery is safe enough. No specific concern will arise
Desalting	Internal flooding	Disposal water leakage	S = 1 $F = 1$ $D = 3$ $Total = 3$	Assume the flooding only impacts internally. No public concern raises
Control of plant.	Loss of cooling water	Flares can activate, lots of heat within the power plant	S = 1 F = 2 D = 1 Total = 2	Given that the shutdown of NPP is safe enough. No specific concern will arise
Cooling Tower pond	Degradation of ultimate heat sink	Oil/byproducts detonation at Refinery plant fills the cooling tower pond with debris	S = 1 F = 2 D = 1 Total = 2	Given that the shutdown of NPP is safe enough. No specific concern will arise
External Supply Tanks integrity	Damage to CST, other supply tanks	Oil/byproducts detonation at Refinery plant	S = 1 F = 2 D = 1 Total = 2	Given that the shutdown of NPP is safe enough. No specific concern will arise
Forced air cooling for non-safety buildings	Loss of HVAC equipment	Oil/byproducts detonation at Refinery plant	S = 1 F = 2 D = 1 Total = 2	Given that the shutdown of NPP is safe enough. No specific concern will arise

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Non-Safety Service water pump house	Damage and/or loss of service water building and equipment	Oil/byproducts detonation at Refinery plant	S = 1 $F = 2$ $D = 1$ $Total = 2$	Given that the shutdown of NPP is safe enough. No specific concern will arise
Spray pond	Degradation of ultimate heat sink	Oil/byproducts detonation at Refinery plant fills the spray pond with debris	S = 1 $F = 2$ $D = 1$ $Total = 2$	Given that the shutdown of NPP is safe enough. No specific concern will arise
Steam diversion load roughly 5% thermal	Prompt loss of thermal load	Oil/byproducts detonation at Refinery plant	S = 1 $F = 2$ $D = 1$ $Total = 2$	Given that the shutdown of NPP is safe enough. No specific concern will arise
Stream Quality Improvement and Blending	Fire	Buildup of the flammable vapors	S = 2 F = 1 D = 1 Total = 2	public concern due to the potential impacts on the air quality
NAPTHA storage at plant	Fires	leaks of the NAPTHA	S = 2 F = 1 D = 1 Total = 2	public concern due to the potential impacts on the air quality
Oil storage at plant	Fires	leaks of the oil	S = 2 F = 1 D = 1 Total = 2	public concern due to the potential impacts on the air quality
Hydrocarbon production and storage	Mechanical injuries, can cut, create missiles, bend pipe, etc.	High-pressure steam leak	S = 1 $F = 2$ $D = 1$ $Total = 2$	The concern only valid inside refinery. No public concern
Desalting	Fire	detonation for hydrocarbons	S = 2 $F = 1$ $D = 1$ $Total = 2$	public concern due to the potential impacts on the air quality

Table F-3. Public safety and perception based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Public	General Notes
Stream Quality Improvement and Blending	Fire	Buildup of the flammable vapors	S = 2 $F = 1$ $D = 1$ $Total = 2$	public concern due to the potential impacts on the air quality
NAPTHA storage at plant	Fires	leaks of the NAPTHA	S = 2 $F = 1$ $D = 1$ $Total = 2$	public concern due to the potential impacts on the air quality
Oil storage at plant	Fires	leaks of the oil	S = 2 $F = 1$ $D = 1$ $Total = 2$	public concern due to the potential impacts on the air quality
Desalting	Failures of removing residual water	pumps malfunctions; unsuccessful splits	S = 1 $F = 1$ $D = 2$ $Total = 2$	Assume the flooding only impacts internally. No public concern raises

Table F-4. Economy based FMEA results for Petroleum Refinery Facility

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
Upgrading and Conversion	Fire	Ignition of a buildup of flammable vapors	S = 10 F = 4 D = 1 Total = 40	This is based on two actual events reported in 2005. \$21.1 billion settlement for the victims and their families. The other events happened in a distillation tower in 2006. \$20 million settlement for the victims and their families.
Maintenance	Fire	Ignition of gasoline components; ignition of Naphtha. The root cause of these events come from the human error.	S = 8 F = 4 D = 1 Total = 32	This is based on two actual events reported in CSB. One happened in 1999 during a pipe removal which transports Napthta. Over \$13 million in property damage.
Hydrocarbon production and storage	hazardous chemical release	Leak of NAPTHA (pentane + hexane), (different compounds to catalytic reformer for gasoline)	S = 2 F = 4 D = 3 Total = 24	no additional evidence is provided for the revenue loss
Water contamination	toxic, settles in water, low places,	leaks of refinery products (e.g., H2S) to the water system in NPP	S = 4 F = 3 D = 2 Total = 24	Loss of revenue expected for potentially hindered shutdown and equipment damage for NPP. The control room environmental filtering needs to be capable of protecting the room from all potential customer hazards.
Maintenance	Explosions and Fire	inadvertently directing air inside the regenerator through the reactor and the main column, then into the gas concentration unit. Failure to control the air flow occurred during the shutdown. Husky Superior Refinery did not effectively implement process safety management systems	S = 10 F = 2 D = 1 Total = 20	This is based on an actual events reported in 2018. This incident resulted in \$550 million of on-site and \$110,000 of offsite property damage.
Upgrading and Conversion	Explosions and Fire; toxic HF release	rupture of a steel piping component with high nickel and copper content that had corroded from HF and thinned faster than adjacent piping components with lower nickel and copper content.	S = 10 F = 2 D = 1 Total = 20	This is based on an actual event reported in 2019 at PES refinery in Philadelphia, Pennsylvania. Marsh JLT Specialty reported that the incident resulted in an estimated property damage loss of \$750 million.

Table F-4. Economy based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
Upgrading and Conversion	Fire; toxic chlorine release	temperature control failure; propane vapor release from cracked control station piping	S = 9 F = 2 D = 1 Total = 18	This is based on an actual events reported in 2007. The nearby chlorine container was affected and 2.5 tons of chlorine has been released. Direct losses attributed to the fire were reported to exceed \$50 million
Purging	Fire	Flammable gas leaks from a failed separation vessel where overpressurization happened and no safety mitigation system is available.	S = 8 F = 2 D = 1 Total = 16	This is a real accident happened in Sonat Exploration Company in 1988. the incident resulted in about \$200,000 worth of damage, including the destruction of the third-stage separator, four private vehicles, and a backhoe and damage to the facility storage tanks
Upgrading and Conversion	Explosions and Fire	Heat exchanger rupture due to high temperature hydrogen attack	S = 8 F = 2 D = 1 Total = 16	This is based on an actual events reported in 2010. Moderate property damage from \$500,000 to \$2 million.
External Power to NPP	shutdown loss of revenue	weather	S = 2 F = 7 D = 1 Total = 14	no additional evidence is provided for the revenue loss
External Power to Refinery	shutdown loss of revenue	weather	S = 2 F = 7 D = 1 Total = 14	no additional evidence is provided for the revenue loss
Upgrading and Conversion	Explosions and fires	corrosion of vapor pipeline	S = 7 F = 2 D = 1 Total = 14	This is a real accident happened in Shell plant explosion in Norco, Louisiana in 1988. The explosion released 159 million pounds (72 kt) of toxic chemicals into the air, which led to widespread damage and the evacuating of 4,500 people.
Fractionation	Explosions and Fire	Pipe rupture	S = 6 F = 2 D = 1 Total = 12	Approximately 15000 people from the surrounding area sought medical treatment due to a large plume of particulates and vapor traveling across the area

Table F-4. Economy based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
Sulfuric Acid	Corrosive, can cause leaks in pipes with worse consequences	integrity failure	S = 3 F = 2 D = 2 Total = 12	Lamont refinery accident - destroyed (launched) tower. This may need to the shutdown of the refinery plant. But no specific amount of the dollar value loss specified.
Desalting	Toxic BOC release	leakage of the toxic chemicals/Corrosion of the pipelines	S = 2 F = 1 D = 5 Total = 10	no additional evidence is provided for the revenue loss
External Supply Tanks integrity	Damage to CST, other supply tanks	Oil/byproducts detonation at Refinery plant	S = 5 F = 2 D = 1 Total = 10	Potentially hindered shutdown. NPP would remain offline until tank farm is repaired.
Benzene	Cancerous,	integrity failure	S = 3 F = 1 D = 3 Total = 9	potential shut down of refinery, leading to revenue loss.
H2S storage/use at plant/Contaminant Removal	Toxic H2S release	Leakage of the H2S from storage tanks, pipes	S = 2 F = 1 D = 4 Total = 8	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	High temperature shift converter uses CO and H to create the syngas (used in methane reforming process after CO is created). Issue is carbon oxides need separated to get CO.	Leak of Carbon Monoxide	S = 1 F = 2 D = 4 Total = 8	no additional evidence is provided for the revenue loss
Control of plant	Loss of cooling water	Flares can activate, lots of heat within the power plant	S = 4 F = 2 D = 1 Total = 8	Loss of revenue expected during the shutdown of the NPP.

Table F-4. Economy based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
External Supply Tanks integrity	Damage to CST, other supply tanks	Oil/byproducts detonation at Refinery plant	S = 4 F = 2 D = 1 Total = 8	Loss of revenue expected for potentially hindered shutdown. NPP would remain offline until tank farm is repaired.
Forced air cooling for non-safety buildings	Loss of HVAC equipment	Oil/byproducts detonation at Refinery plant	S = 4 F = 2 D = 1 Total = 8	Loss of revenue expected when NPP operations hindered until repairs are made.
Non-Safety Service water pump house	Damage and/or loss of service water building and equipment	Oil/byproducts detonation at Refinery plant	S = 4 F = 2 D = 1 Total = 8	NPP would need to shut down safely until repairs are made, leading to loss of revenue.
NPP & Refinery administrative support	Damage to staffs' cars, office buildings and equipment	Oil/byproducts detonation at Refinery plant	S = 4 F = 2 D = 1 Total = 8	NPP operations hindered until repairs are made, leading to loss of revenue.
Water contamination	Staff health threat	Contamination by the spill of the Refinery products or feedstocks	S = 4 F = 2 D = 1 Total = 8	Hindered operation of the NPP, leading to loss of revenue.
Water contamination	pH change in intake water	Contamination by the spill of the Refinery products or feedstocks	S = 4 F = 2 D = 1 Total = 8	Need to shut down the NPP unexpectedly due to possible damage to pumps and other equipment, leading to loss of revenue.
Water contamination	Clogging of water intake screens	Contamination by the spill of the Refinery products or feedstocks	S = 4 F = 2 D = 1 Total = 8	Need to shut down the NPP unexpectedly due to clogged intakes,, leading to loss of revenue.
Desalting	Internal flooding	Disposal water leakage	S = 2 $F = 1$ $D = 3$ $Total = 6$	no additional evidence is provided for the revenue loss

Table F-4. Economy based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
Fractionation	Channel Clogged	Buildup of the materials inside the channel	S = 3 $F = 1$ $D = 2$ $Total = 6$	no additional evidence is provided for the revenue loss
HF feedstock	Acidic, bone-seeker	integrity failure	S = 3 $F = 2$ $D = 1$ $Total = 6$	potential shut down of refinery, leading to revenue loss.
Hydrocarbon production and storage	Mechanical injuries, can cut, create missiles, bend pipe, etc.	High-pressure steam leak	S = 2 $F = 1$ $D = 3$ $Total = 6$	potential shut down of refinery, leading to revenue loss.
Hydrocarbon production and storage	hazardous chemical release	Leak of jet fuel	S = 2 $F = 1$ $D = 3$ $Total = 6$	no additional evidence is provided for the revenue loss
Oil storage at plant	potential chemical releases	leaks of the oil	S = 2 $F = 1$ $D = 3$ $Total = 6$	no additional evidence is provided for the revenue loss
Upgrading and Conversion	Explosions and fires	under investigation	S = 3 $F = 2$ $D = 1$ $Total = 6$	This is a real accident happened in Shell Oil refinery in 1989. The fire burn out for three hours and may lead to the shutdown of the refinery plant. Two Shell contract employees were injured. Neighborhoods were not being evacuated.
Cooling Tower pond	Degradation of ultimate heat sink	Oil/byproducts detonation at Refinery plant fills the cooling tower pond with debris	S = 3 $F = 2$ $D = 1$ $Total = 6$	NPP may have to shut down, depending on the severity of the debris, leading to loss of revenue.
Spray pond	Degradation of ultimate heat sink	Oil/byproducts detonation at Refinery plant fills the spray pond with debris	S = 3 F = 2 D = 1 Total = 6	NPP may have to shut down, depending on the severity of the debris and result in revenue loss.

Table F-4. Economy based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
CO2 feedstock	Asphyxiant	leaks of the CO2	S = 1 $F = 1$ $D = 5$ $Total = 5$	no additional evidence is provided for the revenue loss
CO, used as fuel and also as byproduct	Poisonous, asphyxiant	Byproduct of RWGS, also incomplete combustion	S = 1 $F = 1$ $D = 4$ $Total = 4$	no additional evidence is provided for the revenue loss
External Power to NPP	shutdown loss of revenue	fire/detonation	S = 2 $F = 2$ $D = 1$ $Total = 4$	no additional evidence is provided for the revenue loss
Maintenance	Fire	Operation error	S = 2 F = 2 D = 1 Total = 4	no additional evidence is provided for the revenue loss
Upgrading and Conversion	Explosions and Fire	Pipe rupture	S = 2 F = 2 D = 1 Total = 4	no additional evidence is provided for the revenue loss
Upgrading and Conversion	Explosions and Fire	Operation error	S = 2 $F = 2$ $D = 1$ $Total = 4$	no additional evidence is provided for the revenue loss
Upgrading and Conversion	Fire	valve leakage	S = 2 $F = 2$ $D = 1$ $Total = 4$	no additional evidence is provided for the revenue loss
Upgrading and Conversion	Explosions and Fire	Operations error, human factors.	S = 2 $F = 2$ $D = 1$ $Total = 4$	no additional evidence is provided for the revenue loss

Table F-4. Economy based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
Physical protection	Damage to intrusion sensors, or triggering multiple false alarms	Oil/byproducts detonation at Refinery plant	S = 2 F = 2 D = 1 Total = 4	Lowered security posture. Impacted security but not directly affecting the nuclear safety
Primary loop transport of process steam	Pipe Rupture after MSIV, Damage to turbine building equipment, possibly safety power buses, depending on the plant	Corrosion due to chemical release from the refinery plant	S = 1 F = 2 D = 2 Total = 4	Corrosives would take some time to affect the piping. Regular inspection could detect and prevent the problem.
Hydrocarbon production and storage	hazardous chemical release	Leak of vacuum gasoil (BP 700F to 1000F)	S = 1 F = 1 D = 3 Total = 3	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of asphalt (can crack it into some other streams, will autoignite)	S = 1 F = 1 D = 3 Total = 3	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of coke for burning	S = 1 F = 1 D = 3 Total = 3	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of methane	S = 1 F = 1 D = 3 Total = 3	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of butylene	S = 1 $F = 1$ $D = 3$ $Total = 3$	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of iso-butylene	S = 1 F = 1 D = 3 Total = 3	no additional evidence is provided for the revenue loss

Table F-4. Economy based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
Hydrocarbon production and storage	hazardous chemical release	Leak of propane	S = 1 $F = 1$ $D = 3$ $Total = 3$	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of butane	S = 1 $F = 1$ $D = 3$ $Total = 3$	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of ethane	S = 1 $F = 1$ $D = 3$ $Total = 3$	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of propylene	S = 1 $F = 1$ $D = 3$ $Total = 3$	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of alkylation	S = 1 $F = 1$ $D = 3$ $Total = 3$	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of diesel fuel	S = 1 $F = 1$ $D = 3$ $Total = 3$	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of iso-butane	S = 1 $F = 1$ $D = 3$ $Total = 3$	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of heptane and cyclo- hexane byproduct	S = 1 $F = 1$ $D = 3$ $Total = 3$	no additional evidence is provided for the revenue loss

Table F-4. Economy based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
Desalting	Fire	detonation for hydrocarbons	S = 2 $F = 1$ $D = 1$ $Total = 2$	no additional evidence is provided for the revenue loss
Desalting	Failures of removing residual water	pumps malfunctions; unsuccessful splits	S = 1 $F = 1$ $D = 2$ $Total = 2$	no additional evidence is provided for the revenue loss. Lower bound is specified.
Feedstock (crude oil) Transport by Truck	Fueling accident, toxic chemical release	Leakage during delivery	S = 2 $F = 1$ $D = 1$ $Total = 2$	no additional evidence is provided for the revenue loss
Hydrocarbon production and storage	hazardous chemical release	Leak of sour water	S = 1 $F = 1$ $D = 2$ $Total = 2$	no additional evidence is provided for the revenue loss
NAPTHA storage at plant	Fires	leaks of the NAPTHA	S = 2 F = 1 D = 1 Total = 2	no additional evidence is provided for the revenue loss
NAPTHA storage at plant	potential chemical releases (toxicity)	leaks of the NAPTHA	$\begin{aligned} S &= 2 \\ F &= 1 \\ D &= 1 \\ \textbf{Total} &= 2 \end{aligned}$	no additional evidence is provided for the revenue loss
Oil storage at plant	Fires	leaks of the oil	S = 2 F = 1 D = 1 Total = 2	no additional evidence is provided for the revenue loss
Stream Quality Improvement and Blending	Fire	Buildup of the flammable vapors	S = 2 F = 1 D = 1 Total = 2	no additional evidence is provided for the revenue loss

Table F-4. Economy based FMEA results for Petroleum Refinery Facility Continued...

Process Function	Hazard/Effects	Potential Causes/ Mechanisms of Failure	RPN for Economic	General Notes
Critical structure integrity	Damage to critical structures	Oil/byproducts detonation at Refinery plant	S = 1 $F = 2$ $D = 1$ $Total = 2$	Category I buildings are secure to at least 5.0 psig. Safe siting distance will be for 1.0 psig.
Makeup water pipeline	Loss of makeup water supply line	Oil/byproducts detonation at Refinery plant	S = 1 $F = 2$ $D = 1$ $Total = 2$	Would not affect NPP operation. Customer revenue would be lost until repaired.
Primary loop transport of process steam	Pipe Rupture after MSIV Damage to turbine building equipment, possibly safety power buses, depending on the plant	Operational vibration due to the detonation or explosion from the refinery plant	S = 1 $F = 2$ $D = 1$ $Total = 2$	More likely to affect the piping outside of the reboiler room leading to the customer. Prompt loss of heat load would occur.
Spent fuel storage (dry)	Damage to casks causes radiation leak	Oil/byproducts detonation at Refinery plant	S = 1 $F = 2$ $D = 1$ $Total = 2$	Dry casks are rated for fire protection.
Steam diversion load roughly 5% thermal	Prompt loss of thermal load	Oil/byproducts detonation at Refinery plant	S = 1 $F = 2$ $D = 1$ $Total = 2$	Not an issue unless the thermal diversion exceeds 30%.

Page intentionally left blank.

Appendix G: FMEA Results- Pulp and Paper Facility

The FMEA results for a pulp and paper facility are listed in the following tables.

Table G-1. Nuclear power plant based FMEA results for pulp and paper facility.

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
Tertiary water intake contamination	Water supply contamination raises pH to a level that could harm NPP intake and other equipment. Water supply contamination clogs the water intake at the NPP	Chemical leak at paper facility.	S = 5 F = 3 D = 1 Total = 15	Severity is variable upon water supply source. A static source such as a lake or pond could present a hazard to the NPP if there is a leak at the pulp and paper facility. The severity could be reduced if source is a river where the pulp and paper facility is located downstream of the NPP.
Primary loop transport of process steam	Pipe Rupture after MSIV Damage to turbine building equipment, possibly safety power buses, depending on the plant	Explosion at paper facility	S = 1 F = 3 D = 2 Total = 6	Depends on placement. Assume sited at safe distance.
Spent fuel storage (dry)	Cask tip-over due to overpressure, cask structural degradation	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 3	Possible damage to storage building, if used. Facility must have sufficient separation such that dry casks cannot be damaged. Multiple explosions have occurred at pulp and paper facilities, so frequency is a 3.
External Power	Loss of offsite power	Explosion at paper facility that reaches transmission towers	S = 1 F = 3 D = 1 Total = 3	Possible damage to transmission of offsite power. Facility must have sufficient separation such that offsite power cannot be disrupted. Multiple explosions have occurred at pulp and paper facilities, so frequency is a 3. Must also look at next-most fragile components beyond the transmission towers and auxiliary transformers to see if they are sited at critical distances.

 $Table\ G\ 1.\ Nuclear\ power\ plant\ based\ FMEA\ results\ for\ pulp\ and\ paper\ facility\ Continued...$

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
External Supply Tanks integrity	Damage to CST, other supply tanks	Explosion at paper facility that reaches NPP (or flying debris)	S = 1 $F = 3$ $D = 1$ $Total = 3$	Possible damage to storage tank. Facility must have sufficient separation such that dry casks cannot be damaged. Multiple explosions have occurred at pulp and paper facilities, so frequency is a 3.
Makeup water pipeline	Loss of makeup water supply to spray ponds/cooling towers due to damaged pipeline.	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 3	A sufficient supply of makeup water is necessary; a reduction may lead to inadequate cooling. It is presumed that the makeup water pipeline is either underground or enclosed. There is a potential risk of seismic disturbance to the pipeline leading to the ultimate heat sink.
Spray pond	Degradation of ultimate heat sink	Explosion at paper facility	S = 1 $F = 3$ $D = 1$ $Total = 3$	Debris clogging pond, possibly avoidable with proper placement.
Cooling tower pond	Degradation of ultimate heat sink	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 3	Debris clogging pond, possibly avoidable with proper placement.
Non-Safety Service water pump house	Damage and/or loss of service water building and equipment.	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 3	NPP would need to shut down safely until repairs are made.
Forced air cooling for non- safety buildings	Damage and/or loss of NPP building HVAC equipment. Reactor building, admin building, etc.	Explosion at paper facility	S = 1 $F = 3$ $D = 1$ $Total = 3$	Can affect human operations. May have to shut down reactor.
NPP and paper administrative support	Damage to staffs' cars, office buildings and equipment	Explosion at paper facility	S = 1 $F = 3$ $D = 1$ $Total = 3$	While not directly related to NPP safety, damage to support buildings can affect operations. Explosions have spread beyond the boundaries of the pulp and paper mills.
Physical protection	Damage to intrusion sensors, or triggering multiple false alarms, or empty guard posts due to evacuation.	Explosion at paper facility	S = 1 $F = 3$ $D = 1$ $Total = 3$	Lowered physical protection profile increases NPP vulnerability.

Table G-2. Industrial customer based FMEA results for pulp and paper facility

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for Industrial Plant	General Notes
Multiple	Explosion	Multiple (including gas build up in pulp digester after loss of power)	S = 10 F = 3 D = 1 Total = 30	Explosions at pulp and paper mills require a shutdown of operations. In at least one instance an explosion led to a permanent shutdown.
Multiple	Entire facility shutdown	Fire	S = 5 F = 5 D = 1 Total = 25	There are many cases of fires occurring at pulp and paper facilities with a wide range of causes. Fires often lead to a shutdown of operation.
Delignification/washing/bleaching	Chemical Exposure	Leak of Chlorine Dioxide, black liquor, white liquor, etc.	S = 3 F = 4 D = 2 Total = 24	Multiple cases of chemical leaks at paper facilities have been recorded. Chlorine dioxide inhalation has led to death.
Delignification/washing/bleaching	Entire facility shutdown	Leak of Chlorine Dioxide, black liquor, white liquor, etc.	S = 3 F = 4 D = 2 Total = 24	Multiple cases of chemical leaks at paper facilities have been recorded.
Lime Kiln	Natural Gas Exposure	Pipe leak	S = 2 F = 1 D = 2 Total = 4	The lime kiln in current pulp and paper facilities requires the combustion of natural gas. It is possible that this can be eliminated with the use of electric heaters given power from the NPP.
Multiple	Explosion causing damage to nearby houses, other structures, or highway	Multiple (including gas build up in pulp digester after loss of power)	S = 3 F = 2 D = 1 Total = 6	Toxic debris and possible injuries. Explosions have spread beyond the boundaries of the pulp and paper mills.
Debarking/Chipping	Injury to personnel	Multiple (thrown wood chips, saw dust inhalation or eye contact)	S = 1 F = 4 D = 1 Total = 4	OSHA lists multiple accidents within the wood debarking and chipping process.
Bleaching	Chemical Exposure	Leak of hydrogen peroxide	S = 1 F = 1 D = 1 Total = 1	Hydrogen peroxide vapor can lead to eye and throat irritation, or difficulty breathing.

Table G-3. Public safety and perception based FMEA results for pulp and paper facility.

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
Delignification/washing	Contamination of water supply	Leak of Black Liquor, white liquor, etc.	S = 8 F = 4 D = 1 Total = 32	An accident occurred where black liquor leaked from a storage tank and drained into a river leading to the death of approximately 300kg fish.
Multiple	Damage to nearby houses, public buildings, and highway	Explosion at paper facility	S = 8 F = 3 D = 1 Total = 24	Toxic debris and possible injuries. Explosions have spread beyond the boundaries of the pulp and paper mills.
NPP & Pulp Facility administrative support	Damage to staffs' cars, office buildings and equipment	Explosion at paper facility	S = 8 F = 3 D = 1 Total = 24	Operations hindered until repairs are made.
Physical protection	Damage to intrusion sensors, or triggering multiple false alarms	Explosion at paper facility	S = 8 F = 3 D = 1 Total = 24	Lowered physical protection profile increases NPP vulnerability physically and in the eyes of the public.
Delignification/washing/bleaching	Evacuation	Explosion or leak of chlorine dioxide, black liquor, white liquor, etc.	S = 8 F = 2 D = 1 Total = 16	A leak that leads to an evacuation would likely have vastly negative effect on public perception if the leak is near an NPP.

Table G-4. Economy based FMEA results for pulp and paper facility.

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
Bleaching	Toxic exposure to pulp and paper mill employees	Leak of Chlorine Dioxide	S = 2 F = 3 D = 3 Total = 24	Chlorine dioxide inhalation has led to death.
Tertiary water intake contamination	Contamination of water supply	Leak of Black Liquor, white liquor, etc.	S = 5 F = 3 D = 1 Total = 15	Severity is variable upon water supply source. A static source such as a lake or pond could present a hazard to the NPP if there is a leak at the pulp and paper facility. The severity could be reduced if source is a river where the pulp and paper facility is located downstream of the NPP.
Multiple	Explosion causing damage to nearby houses, other structures, or highway. Potential toxic exposure to public.	Multiple (including gas build up in pulp digester after loss of power)	S = 4 F = 3 D = 1 Total = 12	Toxic debris and possible injuries. Explosions have spread beyond the boundaries of the pulp and paper mills.
NPP & paper administrative support	Damage to staffs' cars, office buildings and equipment	Explosion at paper facility	S = 2 F = 3 D = 1 Total = 6	Assumes safe siting distance for the NPP staff but uses severity for pulp and paper staff.
External supply tanks integrity	Damage to CST, other supply tanks	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 6	Possible damage to storage tank. Facility must have sufficient separation such that dry casks cannot be damaged. Multiple explosions have occurred at pulp and paper facilities, so frequency is a 3.
Delignification/washing	Toxic exposure to pulp and paper mill employees	Leak of Black Liquor	S = 2 F = 3 D = 1 Total = 6	Black liquor exposure may cause burns to the skin, eyes, lungs, and upper gastrointestinal tract.
Debarking/Chipping	Injury to personnel	Wood thrown out or workers caught	S = 2 F = 2 D = 1 Total = 4	OSHA lists multiple accidents within the wood debarking and chipping process, but minor effects.
Primary loop transport of process steam	Pipe Rupture after MSIV Damage to turbine building equipment, possibly safety power buses, depending on the plant	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 3	Assumes safe siting distance. Primary loop is essential for heat source of power cycle. No power generation for NPP leads to no basic commodity generation.

Table G 4. Economy based FMEA results for pulp and paper facility Continued \dots

Process Function	Hazard/Effect	Potential Causes/ Mechanisms of Failure	RPN for NPP	General Notes
Makeup water pipeline	Loss of makeup water supply line	Explosion at paper facility	S = 1 $F = 3$ $D = 1$ $Total = 3$	Assumes safe siting distance. Makeup water required for NPP safety, cooling system, would require immediate attention and pause of normal operations.
Non-Safety Service water pump house	Damage and/or loss of service water building and equipment.	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 3	Assumes safe siting distance. NPP would need to shut down safely until repairs are made.
Forced air cooling for non-safety buildings	Loss of HVAC equipment	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 3	Assumes safe siting distance.
External Power	Loss of offsite power	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 3	Assumes safe siting distance. Dependent on emergency power system, how long emergency power is required.
Spray pond	Degradation of ultimate heat sink	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 3	Debris clogging pond, possibly avoidable with proper placement.
Cooling tower pond	Degradation of ultimate heat sink	Explosion at paper facility	S = 1 F = 3 D = 1 Total = 3	Debris clogging pond, possibly avoidable with proper placement.
Physical protection	Damage to intrusion sensors, or triggering multiple false alarms	Explosion at paper facility	S = 1 $F = 3$ $D = 1$ $Total = 3$	Assumes safe siting distance.

Appendix H: Industrial Products and Feedstock Physical Properties for Safety Analysis Supporting Information

Table H-1. Lists of flammable and detonable products and feedstocks in methanol plant [2].

	Flash Points	Auto-ignition Temperature	Flammability Limit
Feedstocks			
Hydrogen	-250°C	400°C	4%-75%
Natural Gas	-161.5°C	537°C	4%-15%
Products			
Methanol	11°C	464°C	6%-37%

Table H-2. Lists of flammable and detonable products and feedstocks in Synthetic Fuel Plant.

	Flash Points	Auto-ignition Temperature	Flammability Limit
Feedstocks			
Hydrogen	-250°C	400°C	4%-75%
Products			
Diesel	>52°C	~257°C	0.60%-6.50%
Jet Fuel	>38°C	~250°C	0.60%-6.00%
Naphtha	>-22°C	~293°C	1.20%-7.00%
Intermediate stream			
Carbon Monoxide	N/A	607°C	10.9%-74.2%

Table H-3. Lists of flammable and detonable products and feedstocks in Refinery Plant.

Streams	Flash Points	Auto-ignition Temperature	Flammability limit
Feedstocks			
Crude Oil	>60°C	N/A	0.70%-7.00%
Hydrogen	-250°C	400°C	4%-75%
Natural Gas	-161.5°C	537°C	4%-15%
Products			
Gasoline	-40°C	>250°C	1.40%-7.60%

Streams	Flash Points	Auto-ignition Temperature	Flammability limit
Liquefied petroleum gas (LPG)	<-40°C	>450°C	2%-11%
Propane	-104°C	450°C	2%
Butane	-60°C	365°C	2%-8%
Jet Fuel	>38°C	~250°C	0.60%-6.00%
Diesel	>52°C	~257°C	0.60%-6.50%
Sulfur	188°C	255°C	N/A
Intermediate streams			
Refinery fuel gas	-188°C	472°C	4%-17%
Naphtha	>-22°C	~293°C	1.20%-7.00%
Atmospheric gasoil	88-99°C	~210°C	1%-6%
Vacuum gasoil	88-99°C	~210°C	1%-6%
Vacuum residue	>100°C	~250°C	N/A. Explosion lower limit =1%-6%
Hydrogen sulfide (H2S)	−82.4 °C	270°C	N/A

Table H-4. Lists of flammable and detonable products and feedstocks in Refinery Plant.

Streams	Flash Points	Auto-ignition Temperature	Flammability upper limit				
Feedstocks							
Wood chips -188°C		537°C	4%-15%				
Black, viscous liquid	>60.5°C	>407°C	0.1%-3.0%				
Products							
Turpentine	35°C	253°C	0.80%				

Table H-5. Lists of toxic products and feedstocks in Methanol Plant

Products	TWA Toxicity	STEL	Oral Toxicity	Dermal Toxicity	
Methanol	ATE= 100 mg/kg	N/A	ATE = 100 mg/kg	ATE = 300 mg/kg	

Table H-6. Lists of toxic products and feedstocks in Synthetic Fuel Plant

	TWA Toxicity	STEL	Dermal Toxicity		
Products		l			
Diesel	N/A	N/A	ATE >5,000 mg/kg	ATE >5,000 mg/kg	
Jet Fuel	N/A	N/A	ATE >5,000 mg/kg (LD50)	ATE >2,000 mg/kg (LD50)	
Naphtha	N/A	N/A	ATE >5,000 mg/kg	ATE >3,350 mg/kg	
Intermediate strea	ım				
Carbon Monoxide	25 ppm (8 hours)- ACGIH	N/A	ATE=1880 ppm	N/A	
	35 ppm (10 hours)- NISOH REL				
	50 ppm (8 hours)- OSHA PEL				

Table H-7. Lists of toxic products and feedstocks in Refinery Plant.

Streams	TWA Toxicity	STEL	Oral Toxicity	Dermal Toxicity				
Feedstocks								
Crude Oil	2,000 mg/m3	N/A	ATE >5,000 mg/m3	ATE >2,000 mg/m3				
Products			•					
Gasoline	100 mg/m3	200 mg/m3	N/A	N/A				
Propane	1,800 mg/m3	N/A	N/A	N/A				
Butane	N/A	1,000 ppm	N/A	N/A				
Jet Fuel	N/A	N/A	ATE >5,000 mg/kg (LD50)	ATE >2,000 mg/kg (LD50)				
Diesel	N/A	N/A	ATE >5,000 mg/kg	ATE >5,000 mg/kg				
Sulfur	N/A	N/A	ATE >2,000 mg/kg	ATE >2,000 mg/kg				
Intermediate	streams		·					
Naphtha	N/A	N/A	ATE >5,000 mg/kg	ATE >3,350 mg/kg				
Atmospheric gasoil	N/A	N/A	ATE >5,000 mg/kg	ATE >2,000 mg/kg				
Vacuum gasoil	N/A	N/A	ATE >5,000 mg/kg	ATE >2,000 mg/kg				
Vacuum residue	N/A	N/A	ATE >5,000 mg/kg	ATE >2,000 mg/kg				
Hydrogen sulfide (H2S)	5 ppm (7 mg/m ³)	10 ppm (14 mg/m ³)	N/A	N/A				
Hydrofluoric acid	N/A	N/A	ATE = 5-50 mg/kg	ATE <50 mg/kg				

Page intentionally left blank.

Appendix I: Gas Component Leakage Frequencies for Safety Analysis Supporting Information

The leak frequencies per year of gas components were calculated in a report for the hydrogen facility analyses by SNL for Reference [1]. Both generic gas and hydrogen-specific components leakage frequencies are listed in this appendix. The hydrogen leak rates were calculated using a Bayesian statistical analysis that combined leak events from non-hydrogen sources that are representative of hydrogen components with the limited data for leak events from hydrogen-specific components. The resulting component leak frequencies are documented as a function of normalized leak size. Further information is included in [1].

Table I-1. Component Leak Frequencies

	Fractional Leak Size	Generic Leak Frequencies (/y)				Hydrogen Leak Frequencies (/y)			
Component		Mean	5th	Median	95th	Mean	5th	Median	95th
	0.0001	6.0E+00	2.5E-01	2.2E+00	1.9E+01	1.0E-01	5.9E-02	1.0E-01	1.6E-01
	0.001	1.8E-01	2.1E-02	1.1E-01	5.4E-01	1.9E-02	6.8E-03	1.7E-02	3.8E-02
Component Compressor Cylinder Filter Hose	0.01	9.2E-03	1.0E-03	5.2E-03	2.7E-02	6.3E-03	1.2E-03	4.6E-03	1.7E-02
	0.1	3.4E-04	8.2E-05	2.6E-04	8.0E-04	2.0E-04	4.6E-05	1.5E-04	4.9E-04
	1	3.3E-05	1.7E-06	1.2E-05	9.3E-05	3.2E-05	2.0E-06	1.5E-05	1.0E-04
	0.0001	1.5E+00	6.6E-02	6.6E-01	5.3E+00	1.6E-06	3.5E-07	1.4E-06	3.4E-06
	0.001	3.4E-02	3.4E-03	2.0E-02	1.0E-01	1.3E-06	3.7E-07	1.2E-06	2.8E-06
Cylinder	0.01	8.4E-04	1.6E-04	6.4E-04	2.1E-03	9.0E-07	2.6E-07	7.9E-07	1.9E-06
	0.1	2.5E-05	6.6E-06	1.9E-05	5.9E-05	5.2E-07	1.6E-07	4.5E-07	1.1E-06
	1	7.6E-07	1.9E-07	6.1E-07	1.8E-06	2.7E-07	8.1E-08	2.3E-07	6.0E-07
	0.0001	6.9E-02	3.4E-04	5.3E-03	8.4E-02	NA	NA	NA	NA
Filter	0.001	1.4E-02	6.2E-04	5.1E-03	4.1E-02	NA	NA	NA	NA
	0.01	1.6E-02	6.0E-04	4.8E-03	3.9E-02	NA	NA	NA	NA
	0.1	6.1E-03	1.4E-03	4.6E-03	1.5E-02	NA	NA	NA	NA
	1	6.4E-03	1.2E-03	4.4E-03	1.6E-02	NA	NA	NA	NA
	0.0001	6.5E-02	1.7E-03	2.0E-02	2.3E-01	NA	NA	NA	NA
Flange	0.001	4.3E-03	3.4E-04	2.2E-03	1.4E-02	NA	NA	NA	NA
	0.01	3.5E-03	8.4E-06	2.4E-04	7.0E-03	NA	NA	NA	NA
	0.1	3.5E-05	8.3E-06	2.7E-05	8.6E-05	NA	NA	NA	NA
	1	1.9E-05	1.9E-07	2.9E-06	4.6E-05	NA	NA	NA	NA
Hose	0.0001	2.8E+01	1.6E+00	1.3E+01	9.4E+01	6.1E-04	2.9E-04	5.8E-04	1.0E-03

Component	Fractional Leak Size	Generic Leak Frequencies (/y)				Hydrogen Leak Frequencies (/y)			
		Mean	5th	Median	95th	Mean	5th	Median	95th
	0.001	2.2E+00	2.9E-01	1.4E+00	6.4E+00	2.2E-04	6.6E-05	2.0E-04	4.5E-04
	0.01	2.1E-01	4.3E-02	1.6E-01	5.2E-01	1.8E-04	5.3E-05	1.6E-04	3.8E-04
	0.1	2.2E-02	6.0E-03	1.7E-02	5.3E-02	1.7E-04	5.1E-05	1.5E-04	3.4E-04
	1	5.6E-03	1.9E-04	2.0E-03	1.8E-02	8.2E-05	9.6E-06	6.2E-05	2.2E-04
	0.0001	1.3E+00	7.0E-02	5.3E-01	4.6E+00	3.6E-05	2.3E-05	3.5E-05	5.1E-05
	0.001	1.7E-01	2.1E-02	1.0E-01	5.2E-01	5.4E-06	8.4E-07	4.7E-06	1.2E-05
Joint	0.01	3.3E-02	4.2E-03	1.8E-02	9.3E-02	8.5E-06	2.9E-06	7.9E-06	1.6E-05
	0.1	4.1E-03	1.3E-03	3.5E-03	8.6E-03	8.3E-06	2.4E-06	7.5E-06	1.7E-05
	1	8.2E-04	2.3E-04	6.3E-04	1.9E-03	7.2E-06	1.8E-06	6.4E-06	1.5E-05
	0.0001	5.9E-04	7.1E-05	3.6E-04	1.8E-03	9.5E-06	2.1E-06	8.0E-06	2.2E-05
	0.001	8.6E-05	1.7E-05	6.2E-05	2.2E-04	4.5E-06	1.1E-06	3.7E-06	1.1E-05
Pipe	0.01	3.5E-05	9.1E-07	1.1E-05	1.3E-04	1.7E-06	9.9E-08	9.6E-07	5.9E-06
	0.1	4.7E-06	2.3E-07	1.9E-06	1.6E-05	8.4E-07	5.8E-08	4.6E-07	2.9E-06
	1	3.7E-06	1.0E-08	3.2E-07	1.0E-05	5.3E-07	5.5E-09	1.5E-07	2.3E-06
	0.0001	3.9E-02	2.4E-03	1.8E-02	1.3E-01	NA	NA	NA	NA
	0.001	6.5E-03	8.5E-04	4.2E-03	1.9E-02	NA	NA	NA	NA
Pump	0.01	2.5E-03	9.9E-05	9.5E-04	8.3E-03	NA	NA	NA	NA
	0.1	2.8E-04	7.2E-05	2.1E-04	6.7E-04	NA	NA	NA	NA
	1	1.2E-04	5.4E-06	4.9E-05	4.1E-04	NA	NA	NA	NA
	0.0001	2.0E-02	2.2E-03	1.2E-02	6.4E-02	2.9E-03	1.9E-03	2.9E-03	4.2E-03
Valve	0.001	2.8E-03	5.0E-04	1.9E-03	7.5E-03	6.3E-04	2.7E-04	5.9E-04	1.1E-03
	0.01	1.2E-03	2.6E-05	3.1E-04	4.0E-03	8.5E-05	6.6E-06	5.4E-05	2.7E-04
	0.1	6.4E-05	1.8E-05	5.3E-05	1.5E-04	3.0E-05	8.7E-06	2.5E-05	6.7E-05
	1	2.6E-05	8.3E-07	8.5E-06	9.1E-05	1.1E-05	4.7E-07	4.8E-06	4.2E-05