



# FPOG Stakeholder Engagement Meeting – Fire Protection Engineering Evaluation (FPEE) Research Update

*Flexible Plant Operations and Generation (FPOG)*

March 19, 2025

0925 -0940 MST

LWRS Project Lead – Jeff Brown

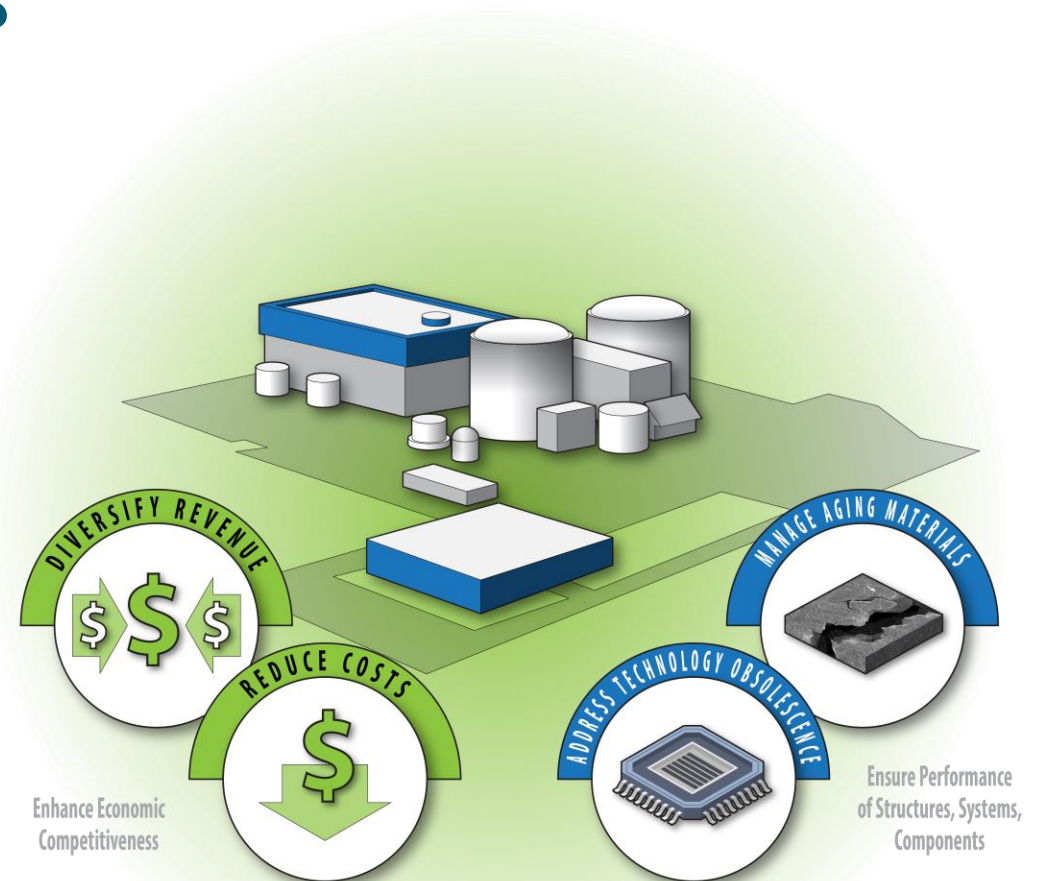
LWRS Pathway Advisor – Jack Cadogan



# Meeting Purpose and Objectives

## Purpose: In-Process Research Update

- Nuclear-integrated Hydrogen—Code and Licensing Separation Distance Considerations (INL\_RPT-24-80476) is under planned revision to:
  - Provide code guidance on utility evaluation of separation distances between co-located high temp electrolysis facilities (HTEF) and NPP SSC's.
  - Obtain expert fire protection engineering review of original code and technical R0 report bases (Jensen Hughes)



21-50005-02

# Assumed Nuclear Integrated Hydrogen Plant Layout

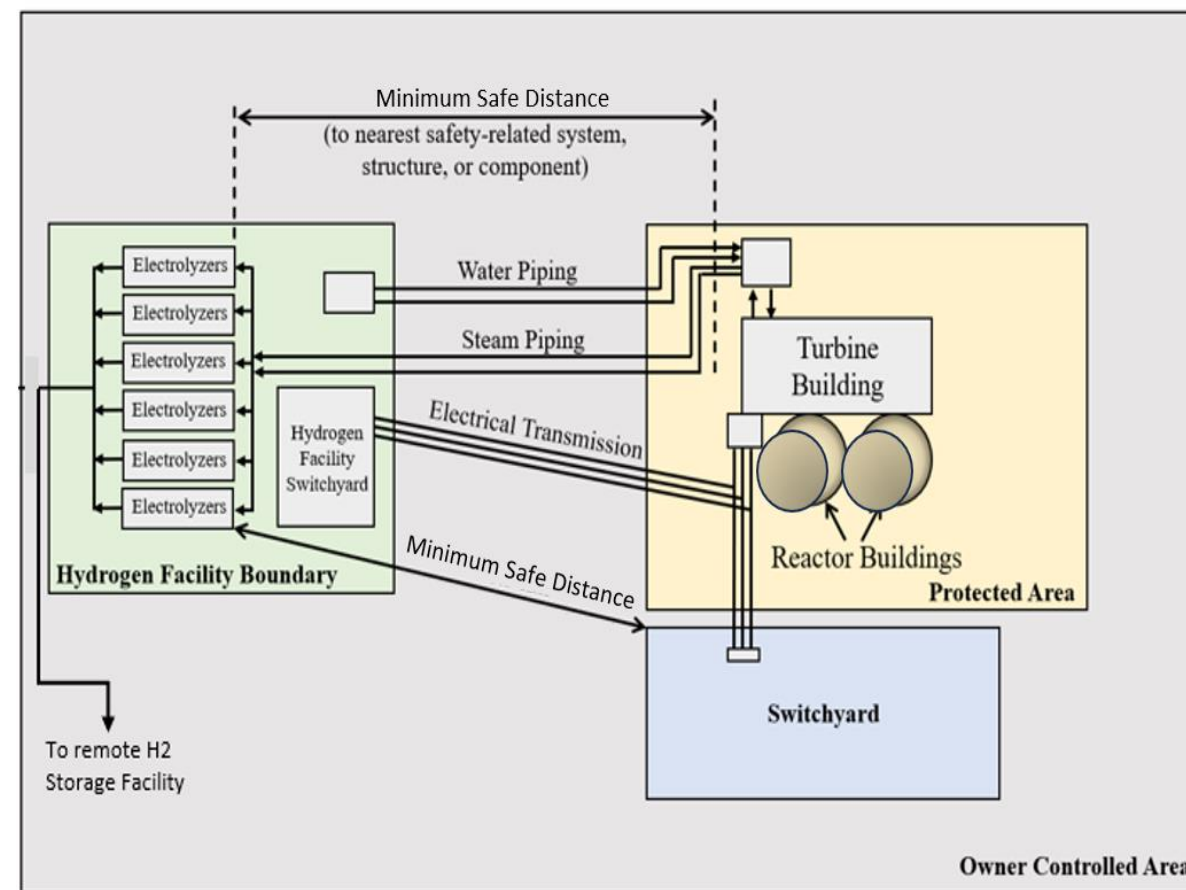
Move this slide to Kurt

## Reference Nuclear Plant

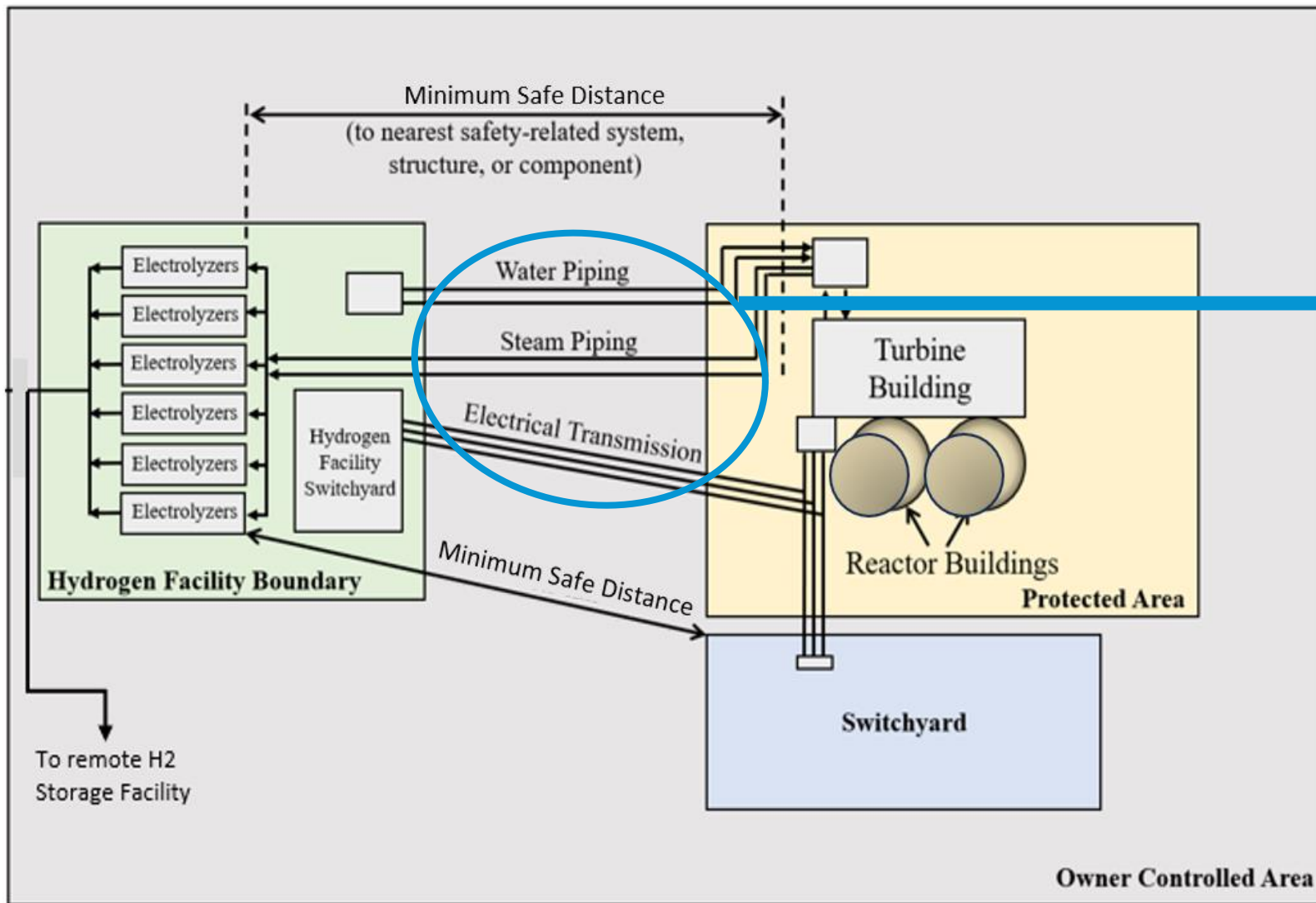
- Westinghouse 4-loop PWR
  - 1200MW<sub>e</sub> / 3,700MW<sub>th</sub> / SWYD: 345kV
  - Typical for 1/3 of operating US NPP Units
- New Hydrogen Steam/Water Supply
- New Behind the Meter Electric

## High Temp Electrolysis (HTE) Facility

- 500MW<sub>DC</sub>
- Thermal Load – 100MW<sub>th</sub>
- Hydrogen Production
  - 300+ tons/day @ 1500 psi



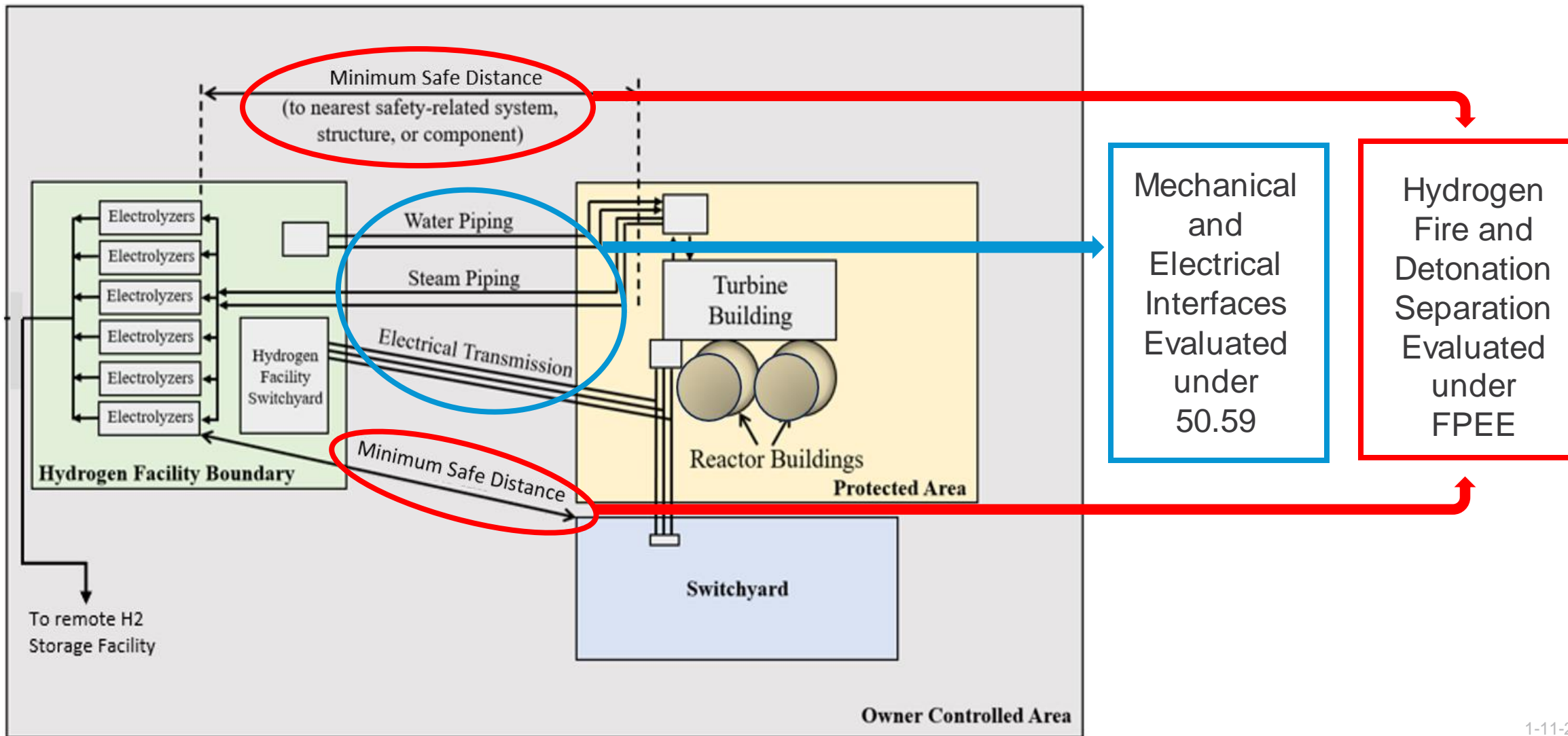
# Plant Interfacing Mods -Licensee Evaluation Approach

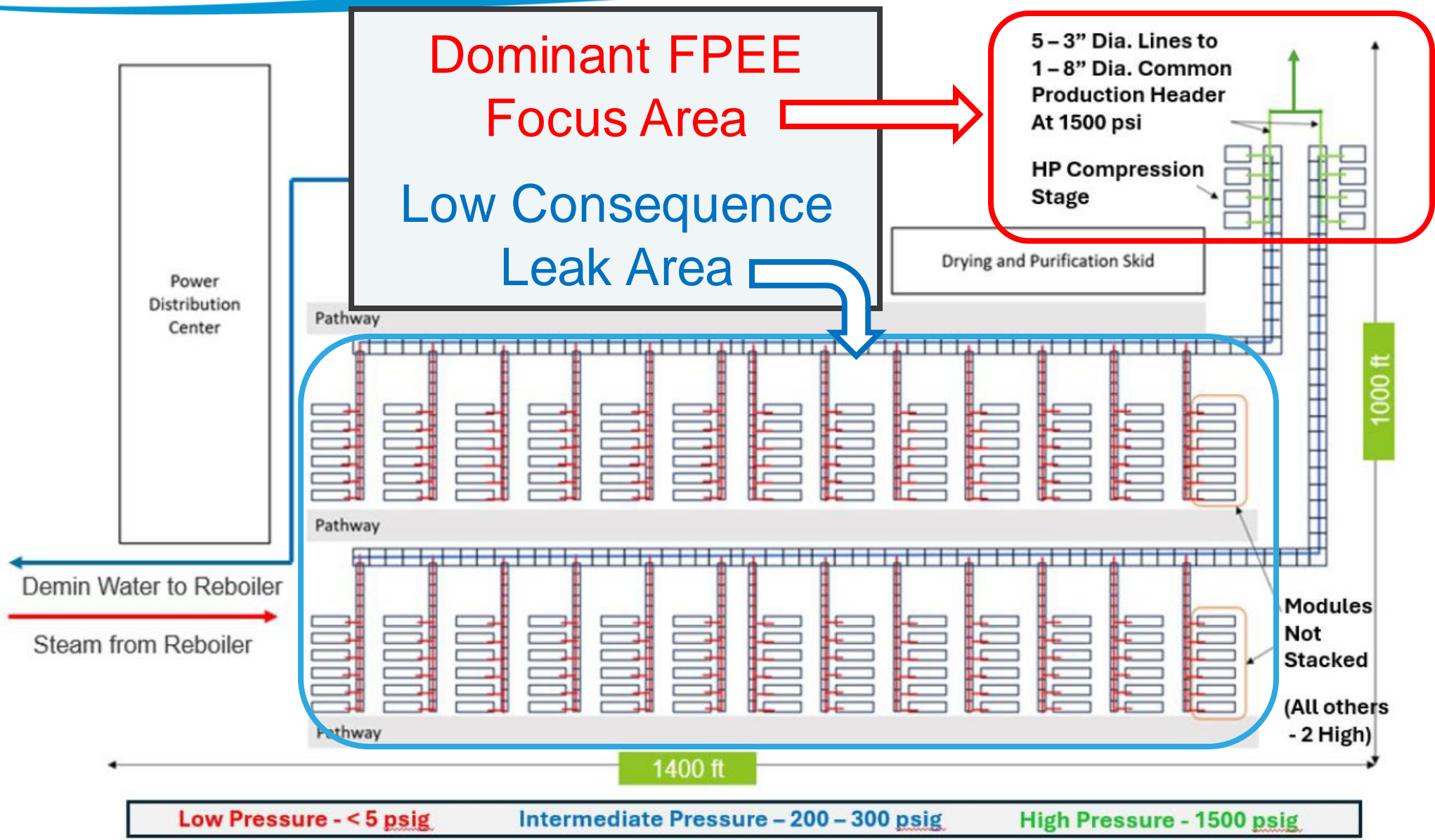


Mechanical and Electrical Interfaces Evaluated under 50.59



# Comparative HTEF Licensee Evaluation Approaches





# Preliminarily Confirmed Research Conclusions

## Nuclear-integrated Hydrogen— Code and Licensing Separation Distance Considerations (INL\_RPT-24-80476)

- NFPA 55 & NFPA 2 code-based evaluation may be used for fire protection program evaluation to determine NPP-to-HTEF separation distances
- Applies under both 10 CFR 50.48(a) and 10 CFR 50.48(c) plant licensing bases



# Preliminarily Confirmed Research Conclusions

## Nuclear-integrated Hydrogen— Code and Licensing Separation Distance Considerations (INL\_RPT-24-80476)

- Alternate non-code elective approaches to separation-distance, as described in previous Idaho National Laboratory research reports may also be conservatively applied.

### Scenario 15

Scenario 15 is a 200.0 mm break with a temperature of 50°C and pressure of 7.0 MPa

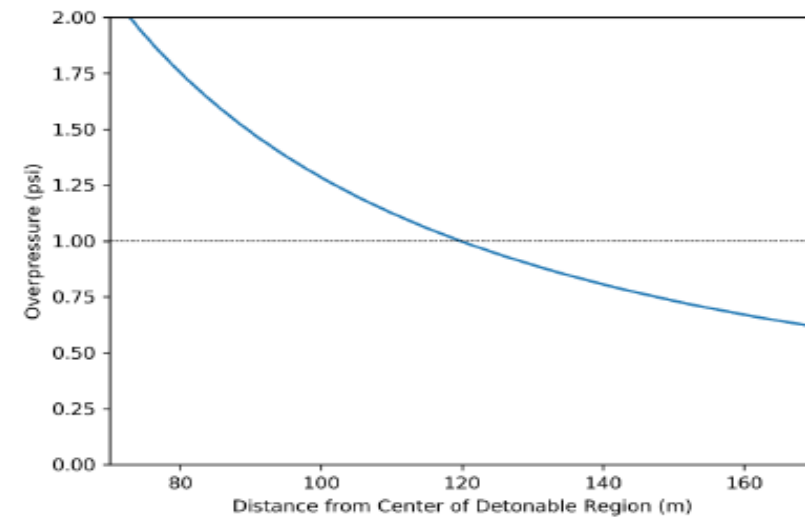
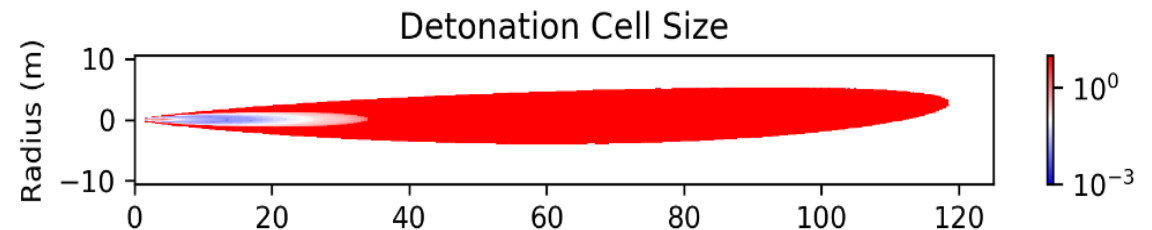
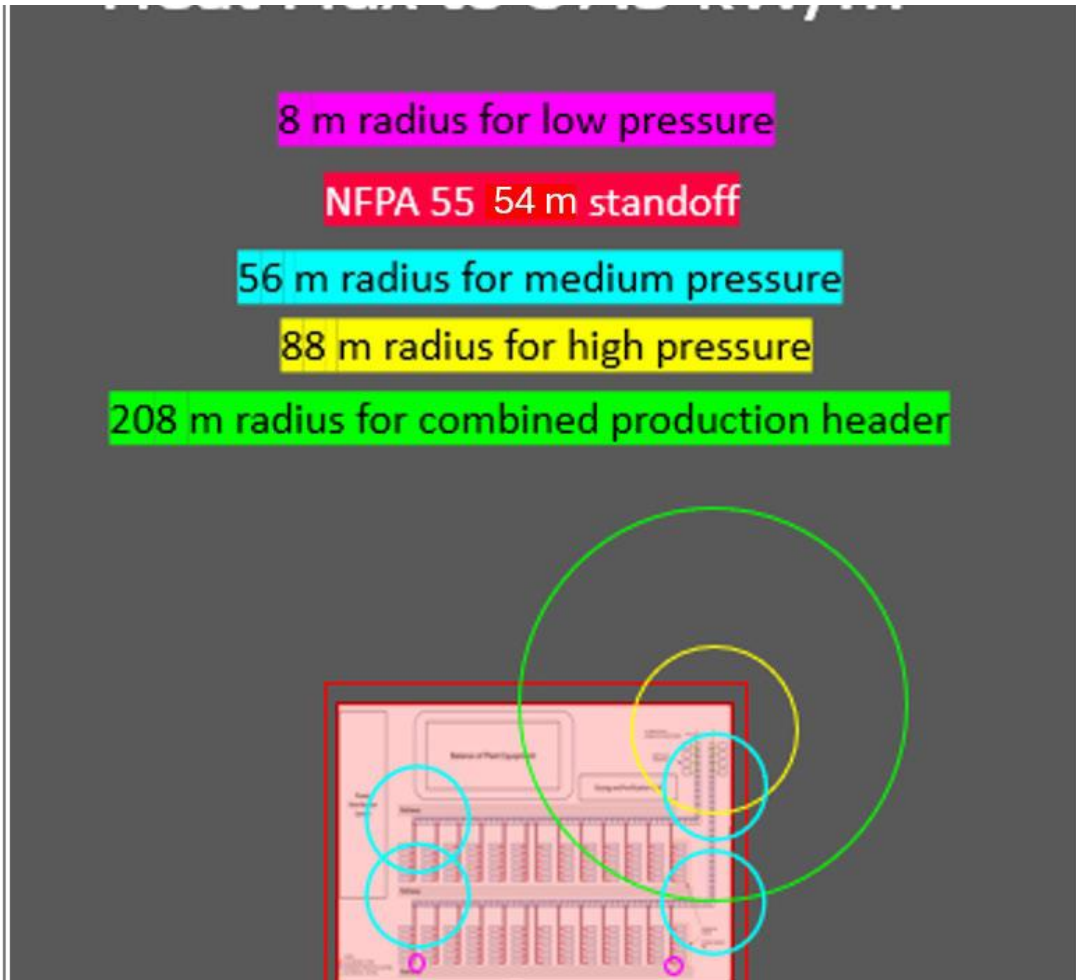


Figure E-15. Scenario 15 Separation Distance Results





# NFPA 55 vs HyRAM+ Heat Flux Separation



	HyRAM+ (@ 37.5 kw/m <sup>2</sup> )	NFPA 55 (Group 3 Exposure)
Leak Size Assumed	76-mm-diameter orifice Case 6 (3-in. pipe)	7.6 mm diameter orifice
Calculated Separation	88 m	21 meters
Leak Size Assumed	203 mm diameter orifice Case 7 (8-in. pipe)	20.3-mm-diameter orifice
Calculated Separation	208 m	54 meters
Bounding Calculated Separation	208 m	54 meters

HyRAM+ heat flux cases run with assumed full pipe guillotine break vs. NFPA 55 assumed 1% diameter orifice leak.

# Bauwens vs. TNT Equiv. O/P Separation

## Bauwens

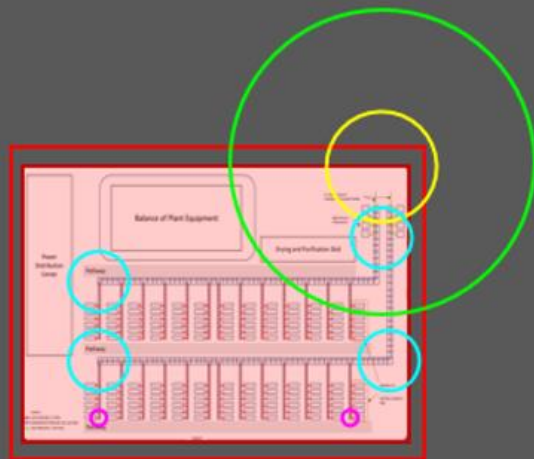
8 m radius for low pressure

NFPA 55 54 m standoff

34 m radius for medium pressure

61 m radius for high pressure

168 m radius for combined production header



## TNT Equivalence

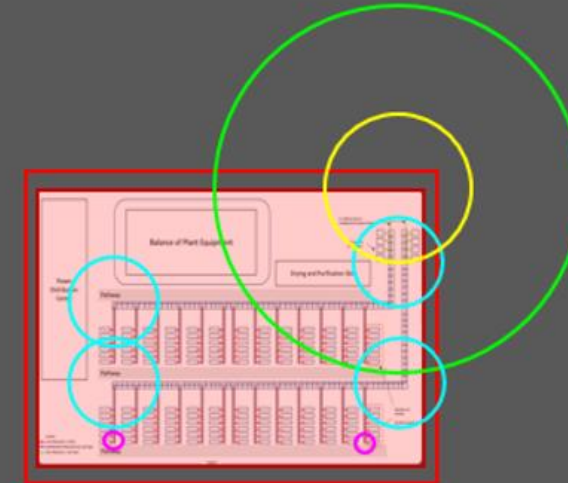
10 m radius for low pressure

NFPA 55 54 m standoff

49 m radius for medium pressure

81 m radius for high pressure

204 m radius for combined production header



## Historical Hydrogen Design Bases at NPPs

- 10 CFR 50.48(a) and 10 CFR 50.48(c) fire protection regulatory frameworks support co-located compressed gaseous hydrogen for plant process use
- Safe NPP-to-H<sub>2</sub> storage separation distances were historically assessed based on NFPA 55, or earlier NFPA 50A, flammable gas regulation rules, depending on plant license vintage
- NFPA 55 (and emerging H<sub>2</sub>-specific NFPA 2):
  - Are based on pressurized hydrogen system leakage manifesting in small leaks, rather than gross failures
  - Code approaches contain significant safety margins



# Sustaining National Nuclear Assets

*[lwrs.inl.gov](http://lwrs.inl.gov)*